

US National Nuclear Data Week 2015

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**Thermal Scattering Law  
Data Generation and Validation  
In the 21<sup>st</sup> Century**

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# Acknowledgement

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- ❑ Past and current undergraduate and graduate students and postdocs at North Carolina State University



- ❑ Collaboration with LLNL and Bettis labs
- ❑ Funding by the NCSP and NR programs

# Vision

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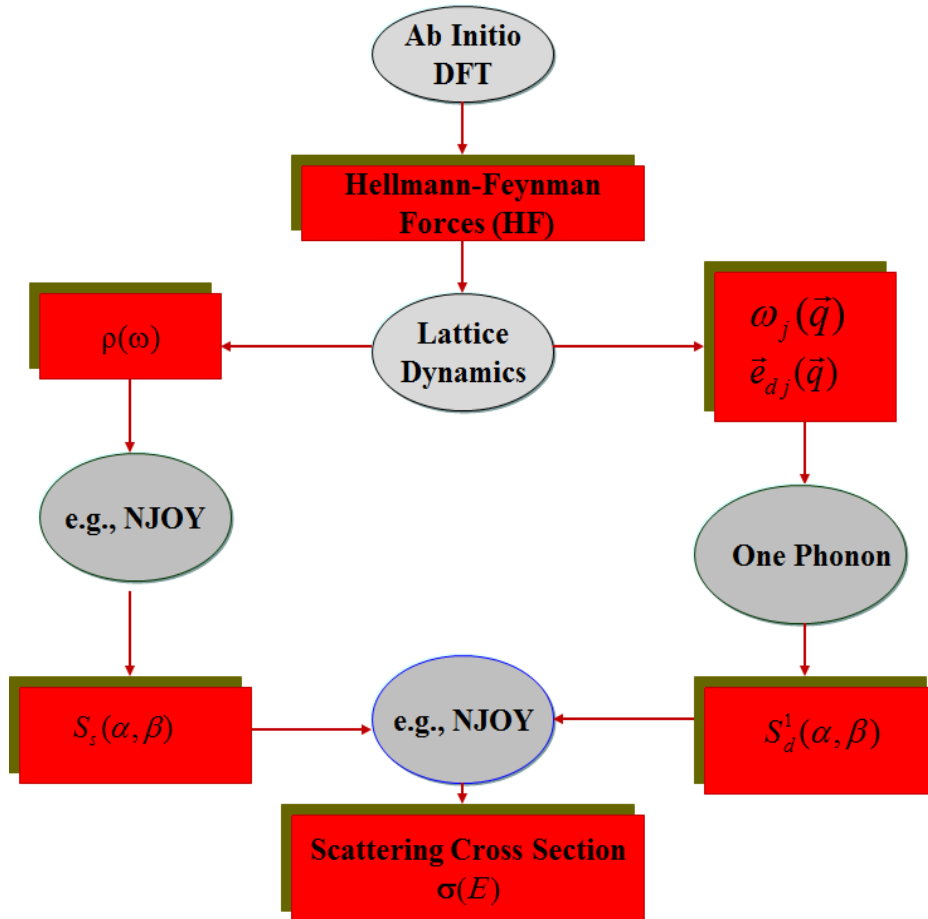
- ❑ Establish a predictive and approximation free approach for generating the needed data (i.e., TSL) to describe the energy exchange of thermal neutrons in matter
  - Initiated in 2000/2001
  
- ❑ Various applications:
  - Nuclear criticality safety
  - Nuclear reactor design
  - Neutron beam spectral shaping (i.e., filtering)
  - Neutron source (cold, ultracold, etc.) characterization

# Methods

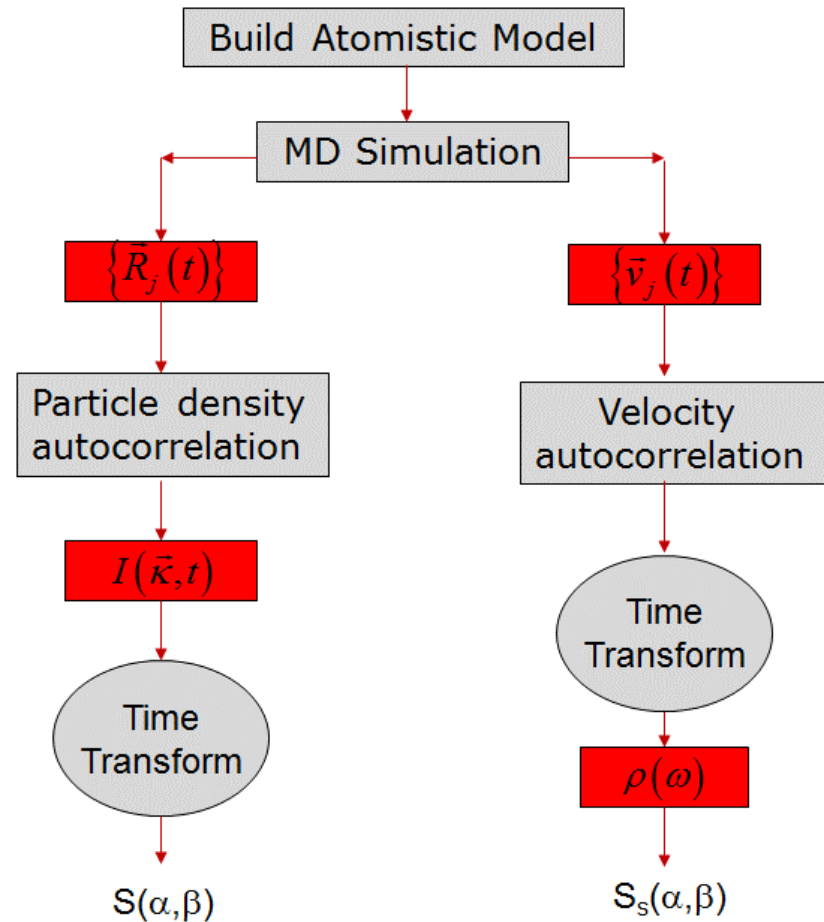
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- $S(\alpha, \beta)$  is a material property
  - can be calculated very accurately!
  
- Several approaches can be used to extract the fundamental information for calculating the scattering law and eventually the cross sections
  - Empirical atomic force analysis combined with dynamical matrix calculations
    - Basis of current ENDF/B libraries
  
  - Ab initio Quantum (DFT) methods combined with dynamical matrix calculations
  
  - Molecular Dynamics (classical MD or ab initio) methods combined with correlation function analysis

## DFT Based

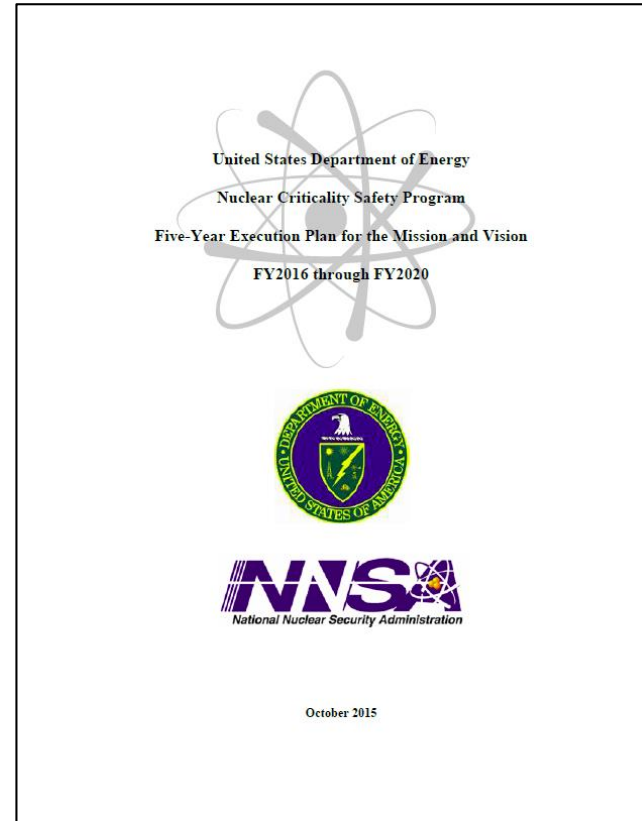
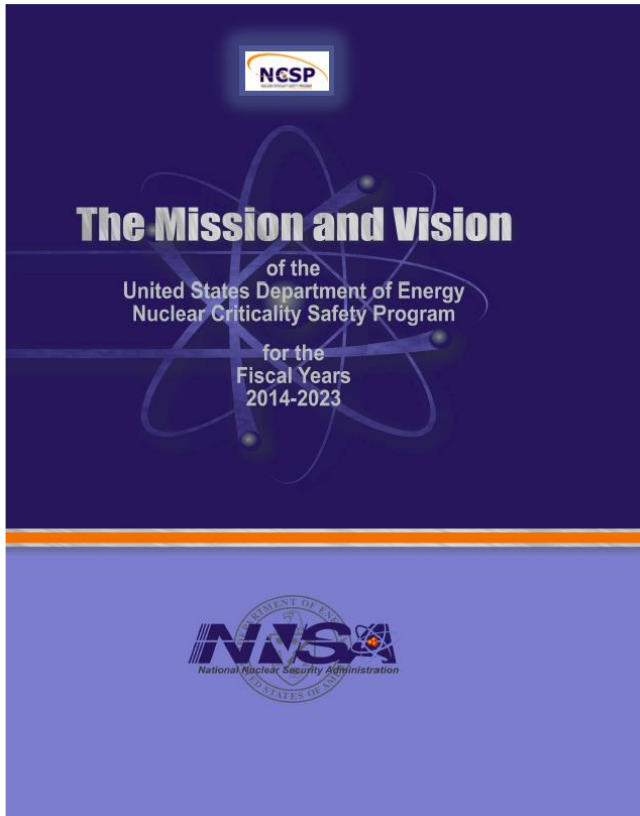


## MD Based



**Merge both into the DFT based Ab Initio MD approach**

# NCSP Program (with NR)



## Appendix B Nuclear Data

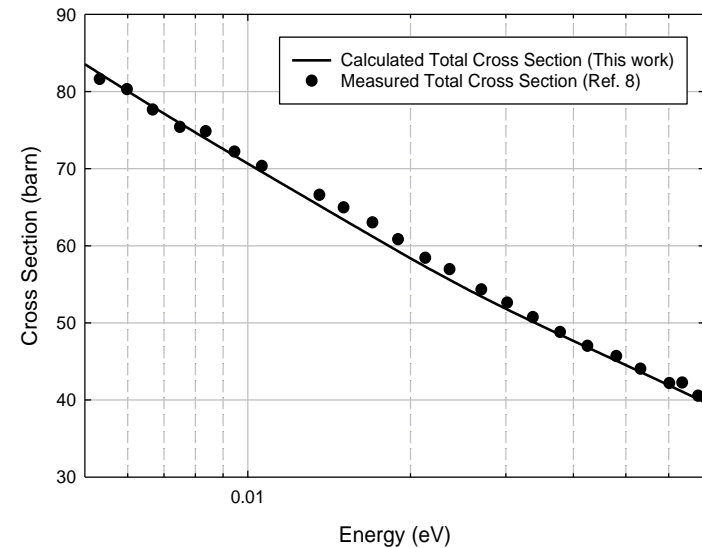
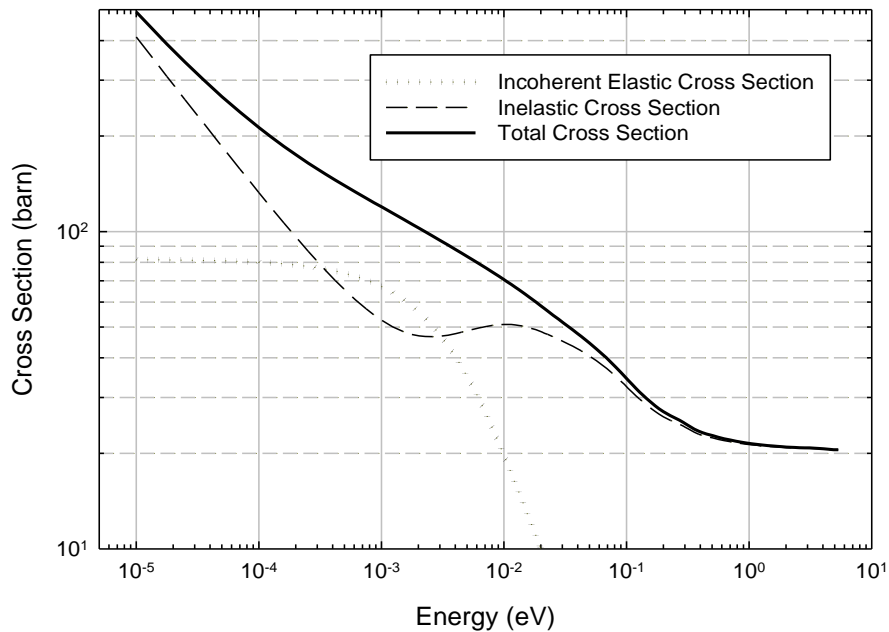
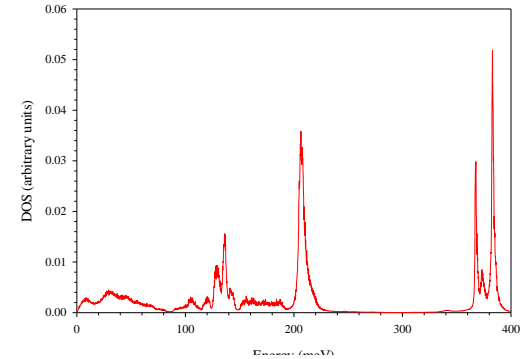
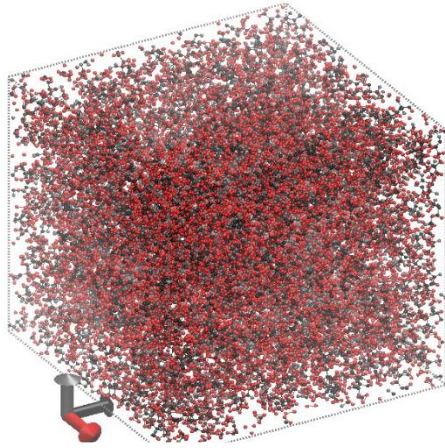
<b>Priority Needs</b> */ Additional Needs	Thermal scattering (BeO, HF, D <sub>2</sub> O, SiO <sub>2</sub> , CH <sub>2</sub> , C <sub>2</sub> F <sub>4</sub> , C <sub>2</sub> O <sub>2</sub> H <sub>4</sub> , etc.), <sup>239</sup> Pu, Cr, <sup>237</sup> Np, Pb, <sup>55</sup> Mn, Ti, <sup>240</sup> Pu / <sup>233</sup> U, Th, Be, <sup>51</sup> V, Zr, F, K, Ca, Mo, Na, La
<b>Completed Evaluations (FY)</b>	Minor Actinides (13), SiO <sub>2</sub> (12), <sup>55</sup> Mn (12), <sup>180,128,183,184,186</sup> W (14)

# Examples - Materials Studied at NCSU

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- ❑ Lucite ( $C_5O_2H_8$ , contributed to NNDC/ENDF, ANS 2015)
- ❑ Polyethylene ( $CH_2$ )
  - Support criticality safety applications
- ❑ Silicon dioxide (contributed to NNDC/ENDF, PHYSOR 2008, ANS 2011, NDS 2014)
  - Support criticality safety applications
- ❑ Silicon carbide (contributed to NNDC/ENDF, ANS 2013)
  - Support advanced fuel cycle applications
- ❑ Graphite, Beryllium (PHYSOR 2004 & 2008, NDS 2014, PHYSOR 2016), BeO (unpublished)
  - Treatment of nuclear graphite (porous system)
  - Including coherent inelastic for both graphite and beryllium
- ❑ uranium-zirconium hydride, calcium hydride, thorium hydride, (PHYSOR 2004), uranium dioxide, uranium silicide
- ❑ Sapphire and bismuth (PHYSOR 2006)
  - Thermal neutron filters
- ❑ Solid methane (predictive analysis – AccApp 2011)

# Lucite ( $C_5O_2H_8$ )

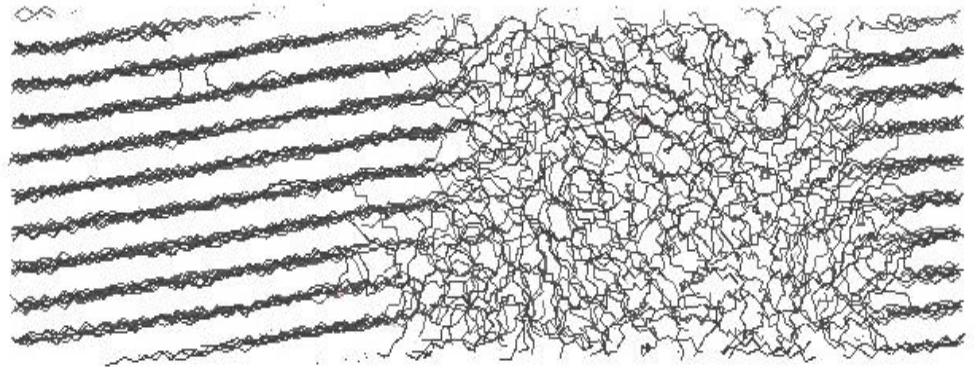
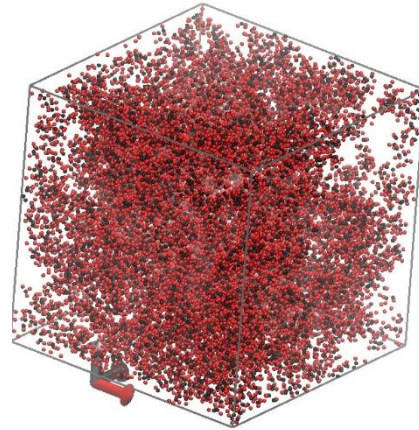


Exp. Data: G. SIBONA et al., Anna. Nucl. Energy, 18, 689 (1991).

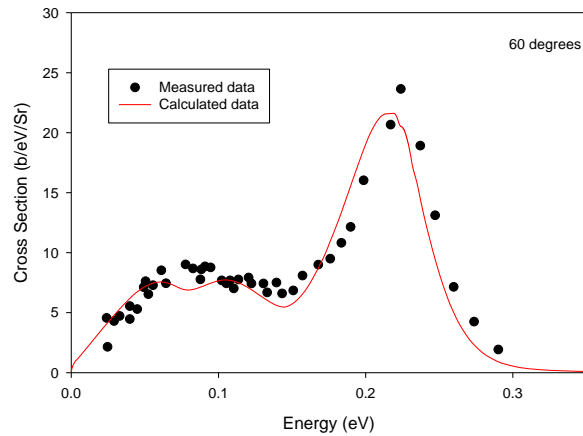
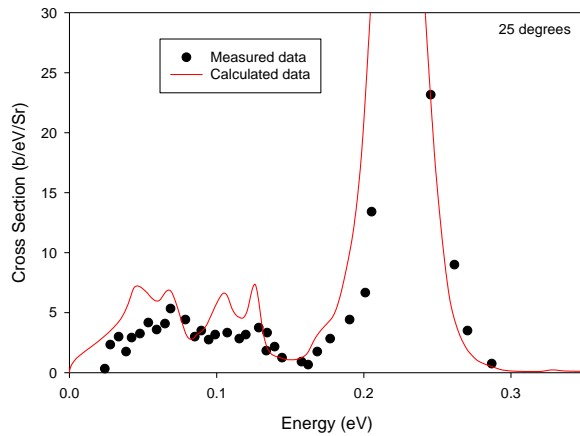
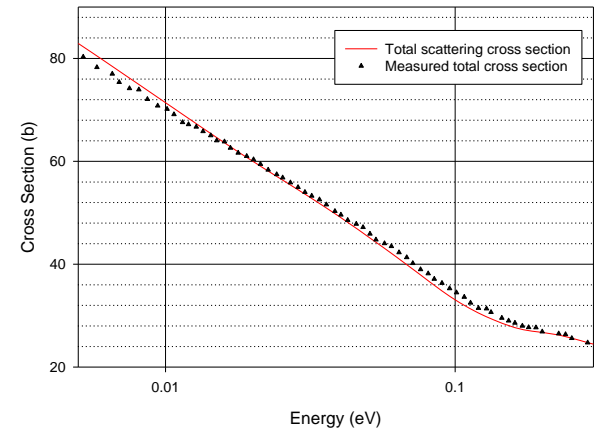
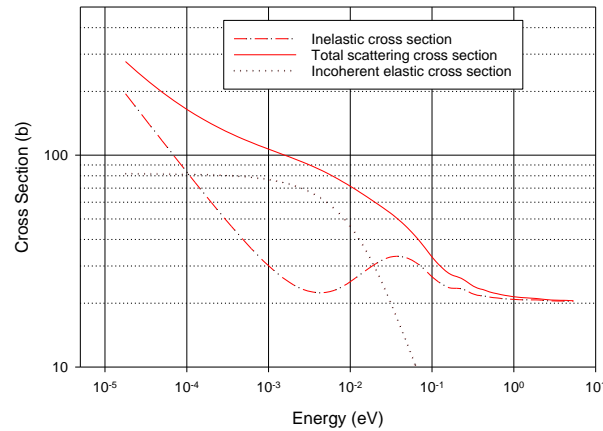
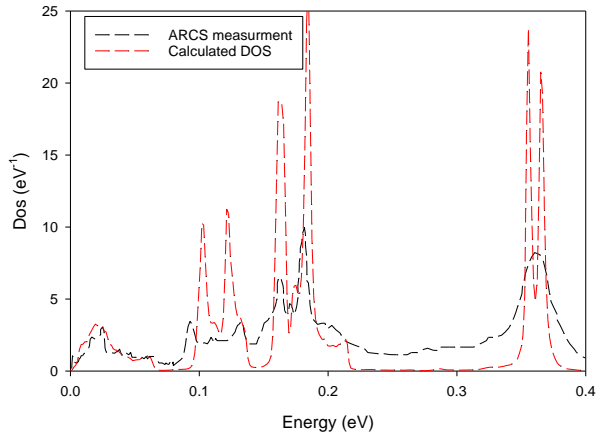


# Polyethylene ( $\text{CH}_2$ ) System

- ❑ Systems of multiple polymer chains are needed to account for inter-chain interactions.
- ❑ Simulated 20 polymer chains, each 200 monomers long, 24,000 atoms.



# Polyethylene (CH<sub>2</sub>)



Exp. Data:

- 1) G. SIBONA et al., Anna. Nucl. Energy, 18, 689 (1991).
- 2) R. E. Hill et al., NIM A, 538, 686 (2005).
- 3) C. M. Lavelle et al., NIM A, 711, 166 (2013).

# TSL Data Evaluation

## Path Forward

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### □ **Simulate/Calculate**

#### ■ **Generate TSL libraries (very efficient)**

- Support applications
- Support design and interpretation of validation and benchmark experiments
- Capture variations in system conditions

### □ **Validate**

- Selected experiments

### □ **Create next generation tools**

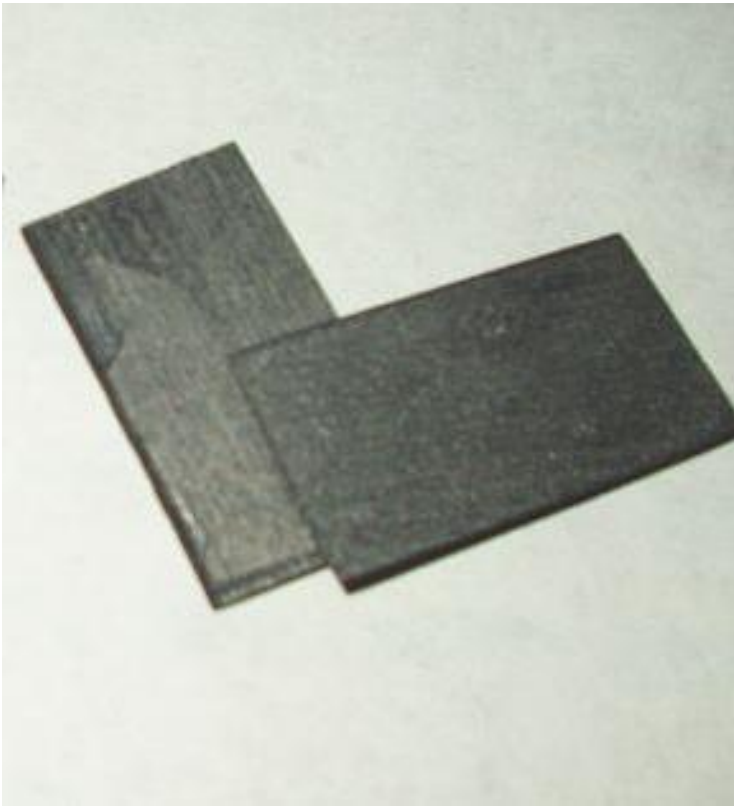
#### ■ **TSL and cross section calculations**

- Take advantage of the wealth of information available through modern atomistic modeling techniques

#### ■ **Uncertainty quantification**

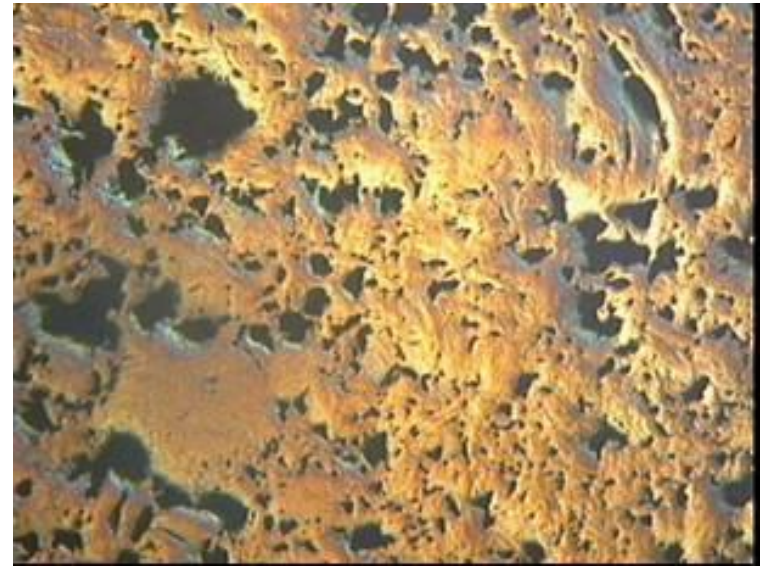
# Graphite Types

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**Ideal Graphite**

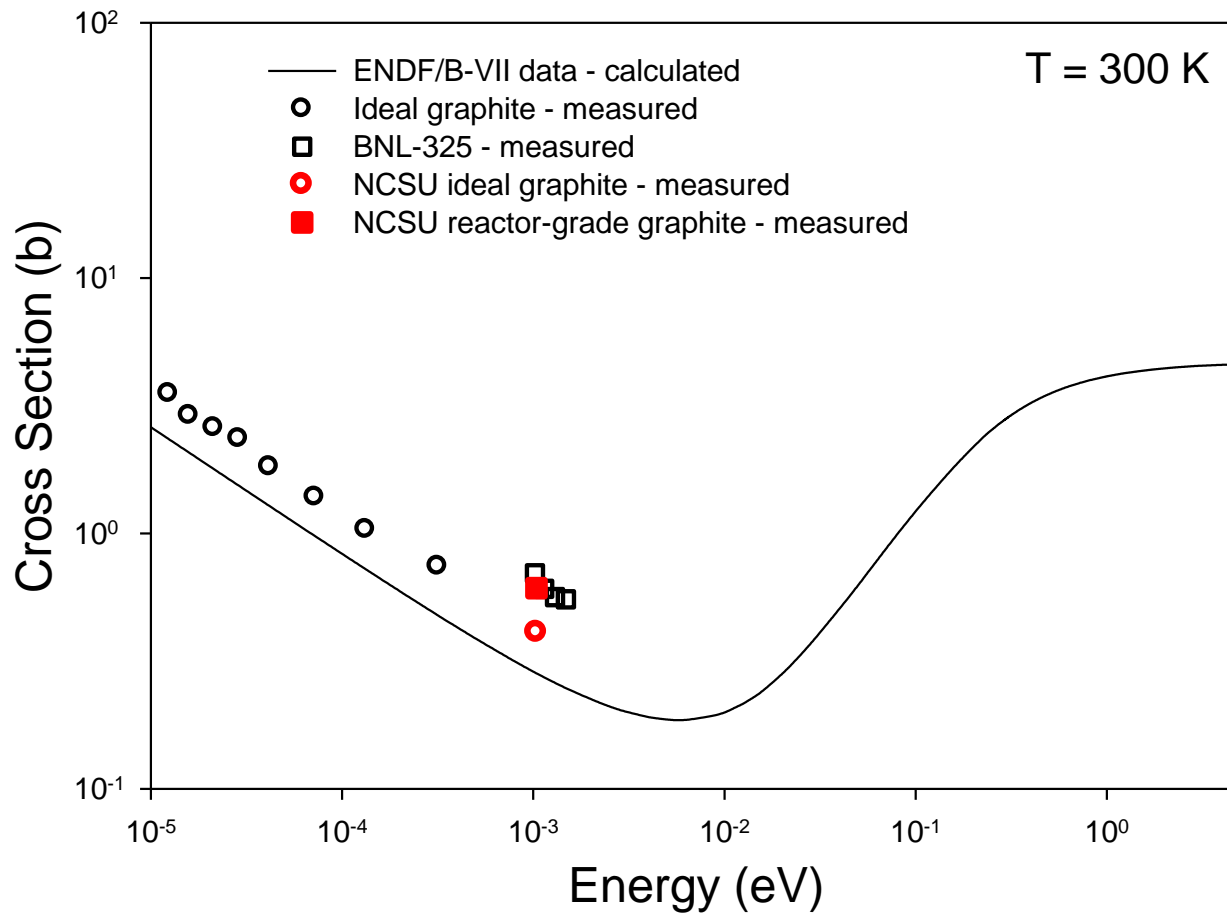
**Density =  $2.25 \text{ g/cm}^3$**



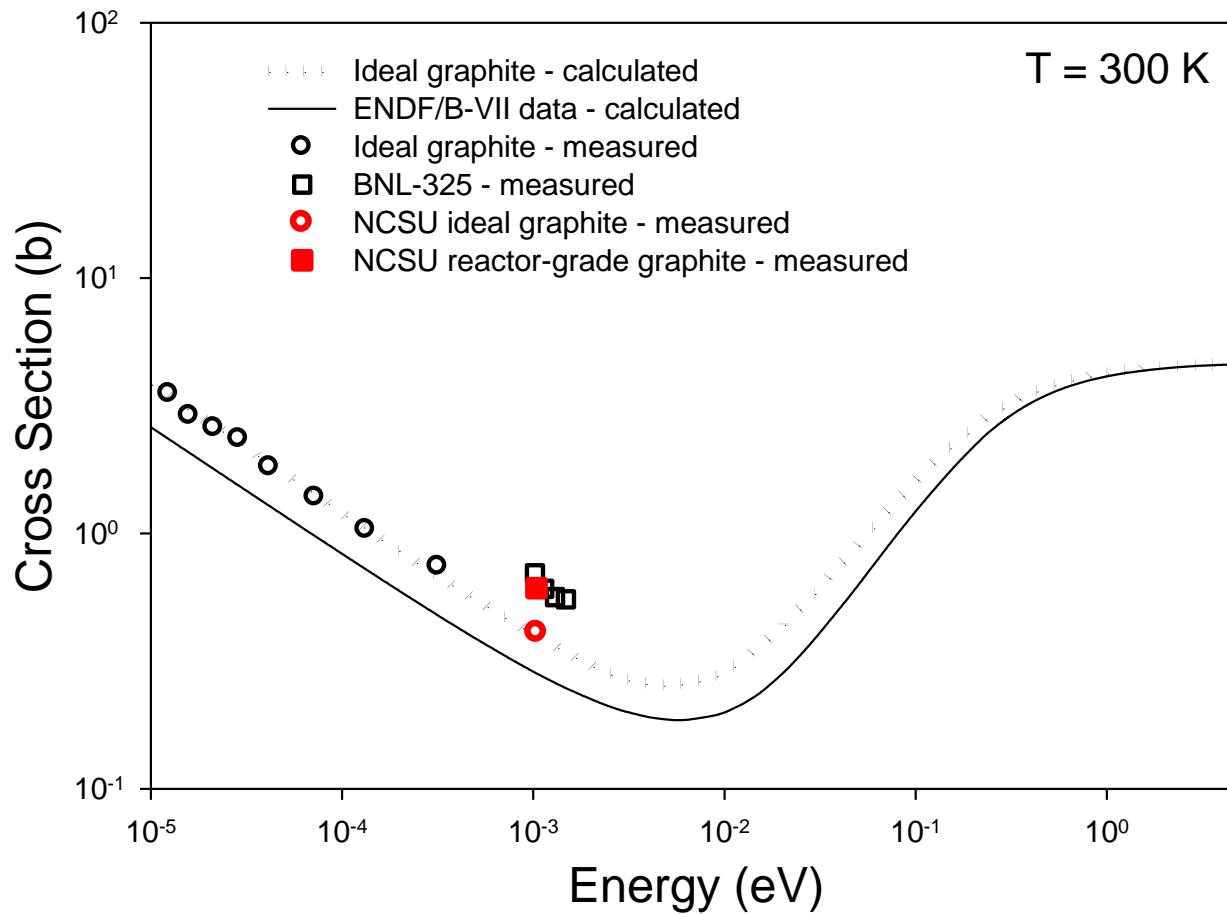
**Nuclear Graphite**

**Density =  $1.5 - 1.8 \text{ g/cm}^3$**

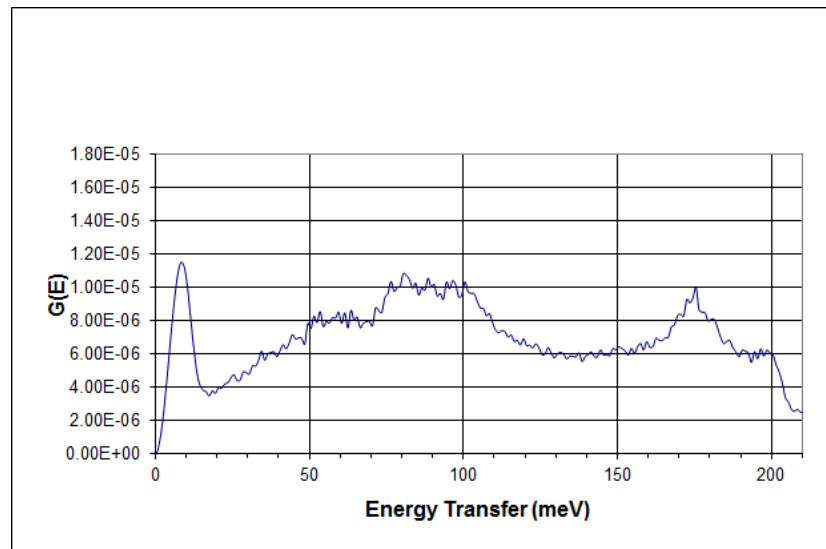
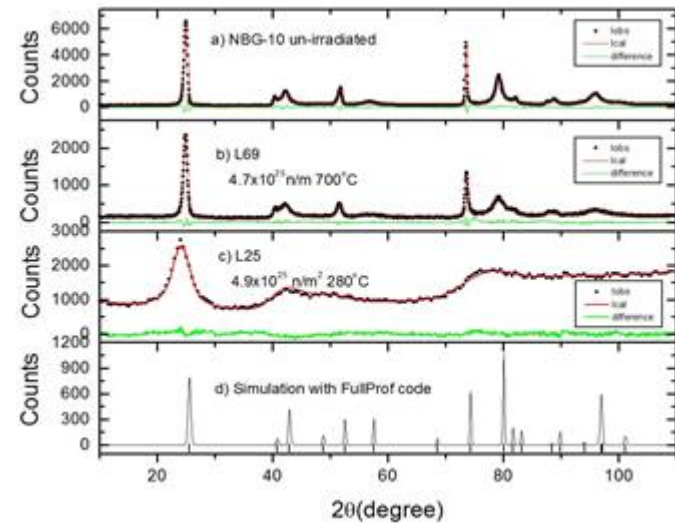
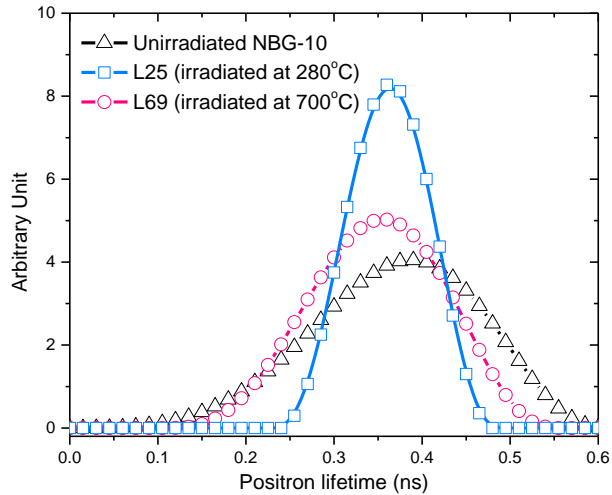
# Graphite



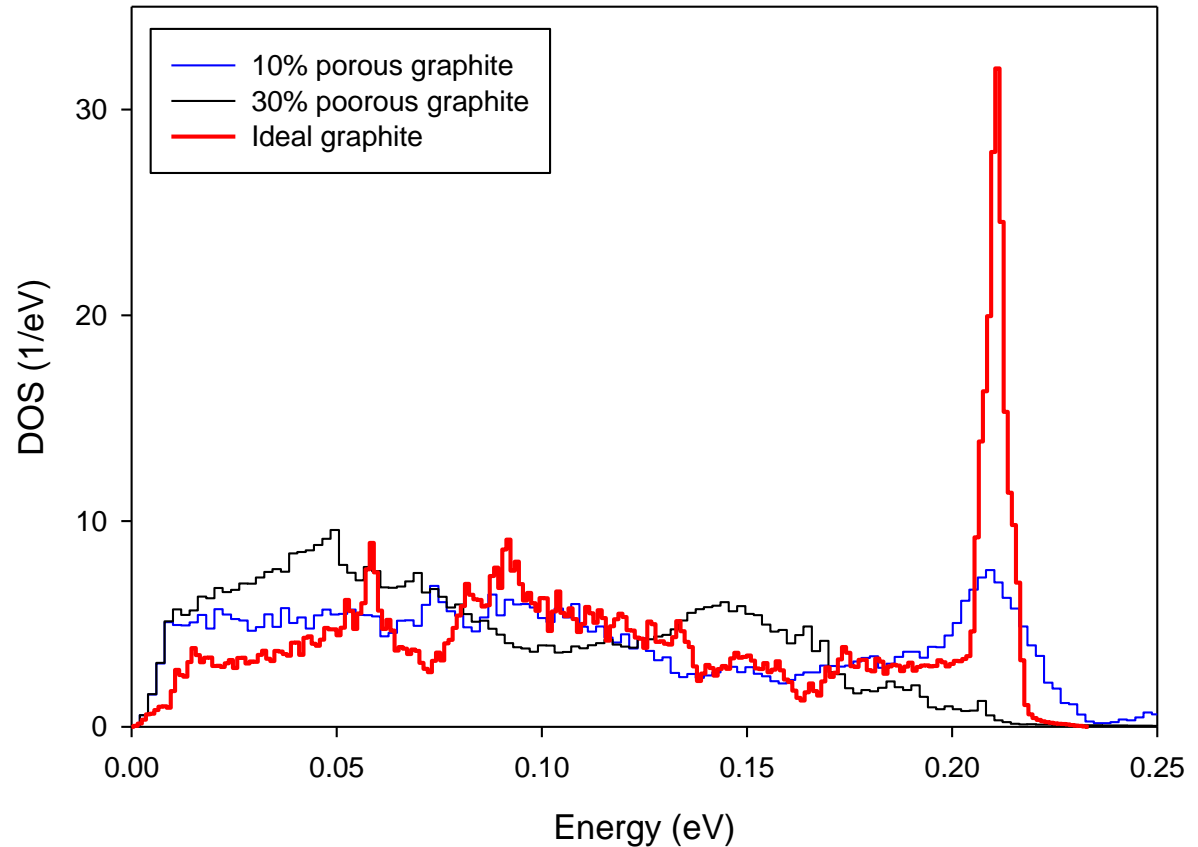
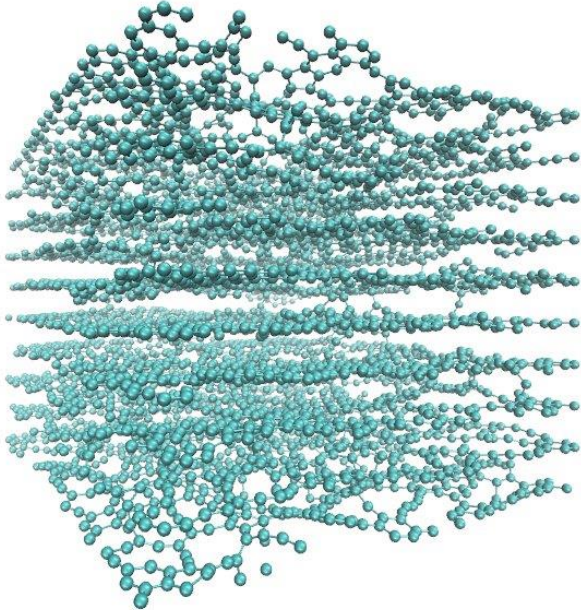
# Graphite



# Targeted Experiments

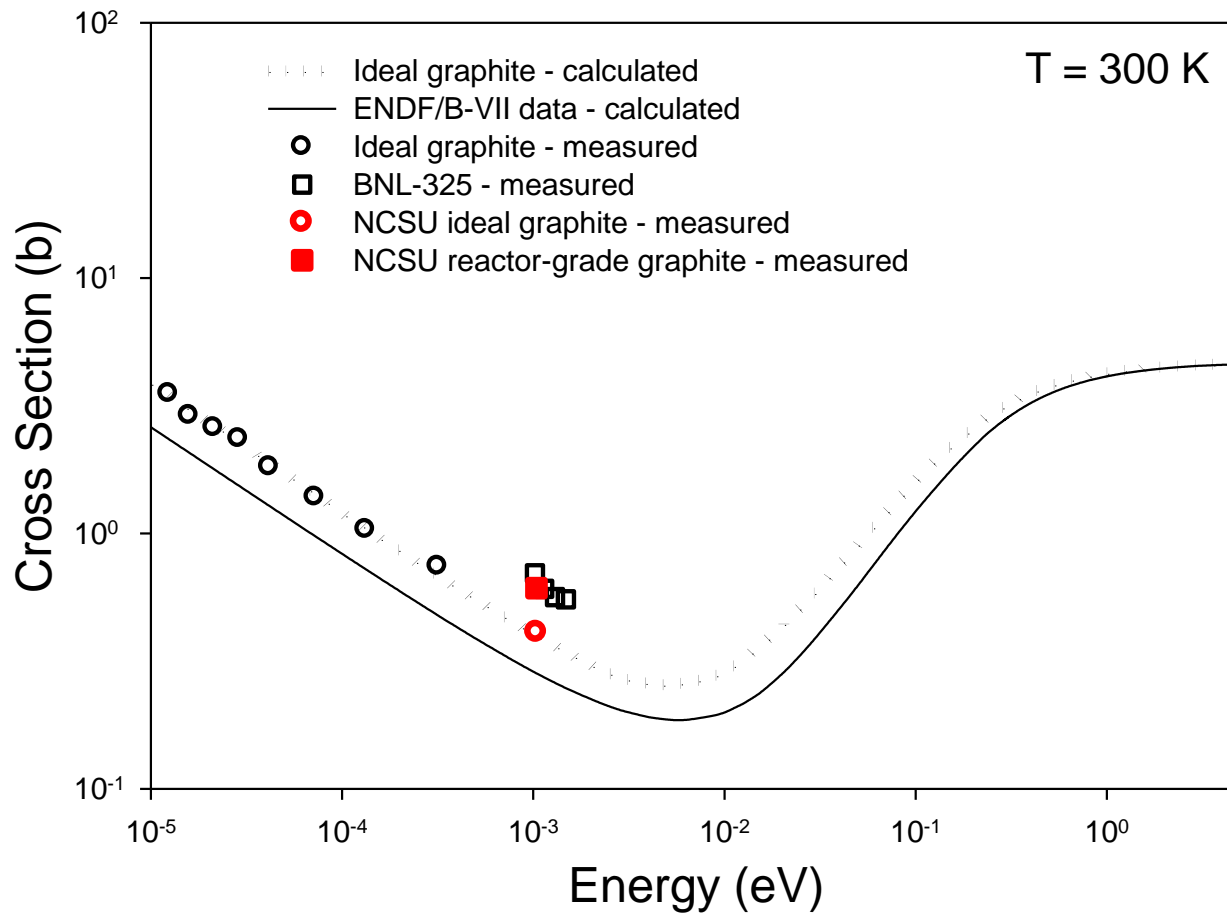


# Molecular Dynamics Models

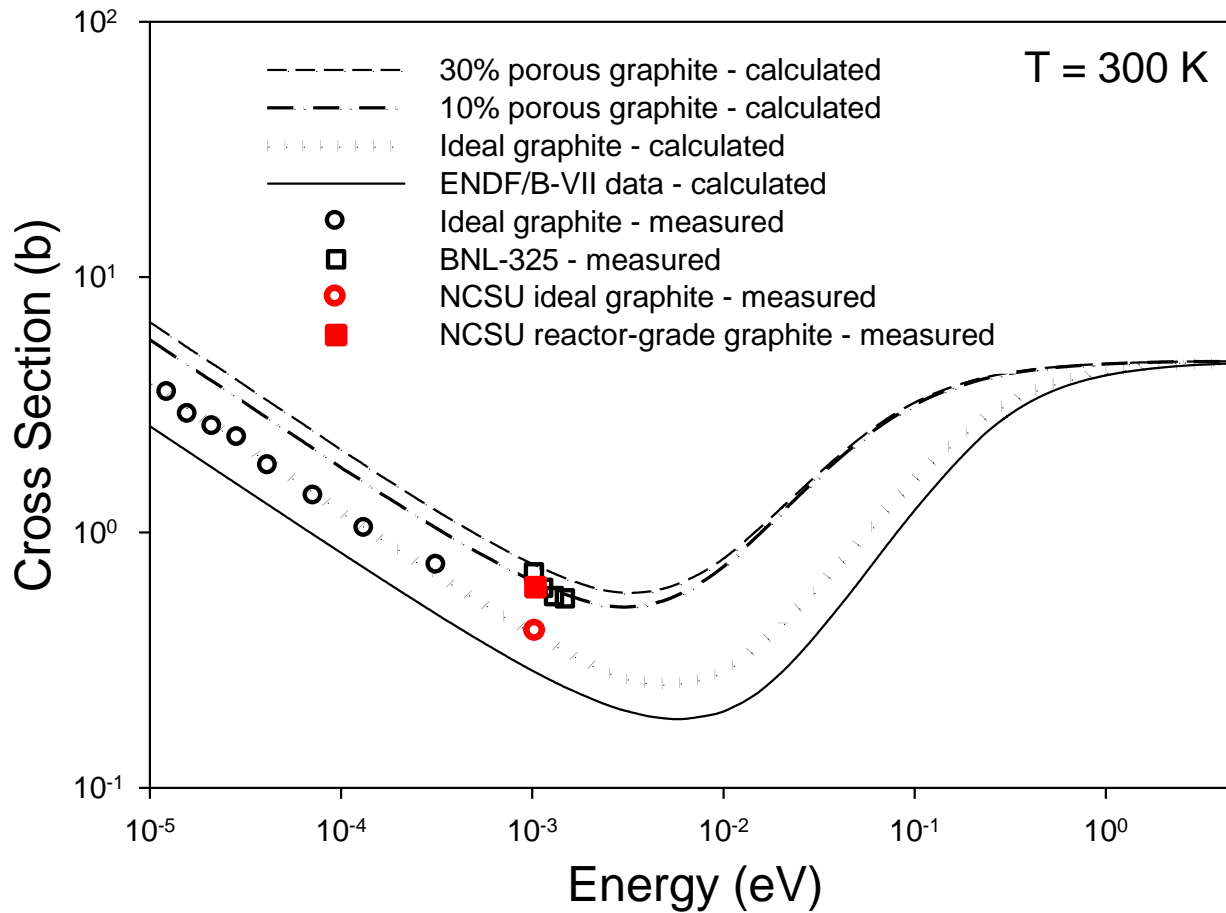




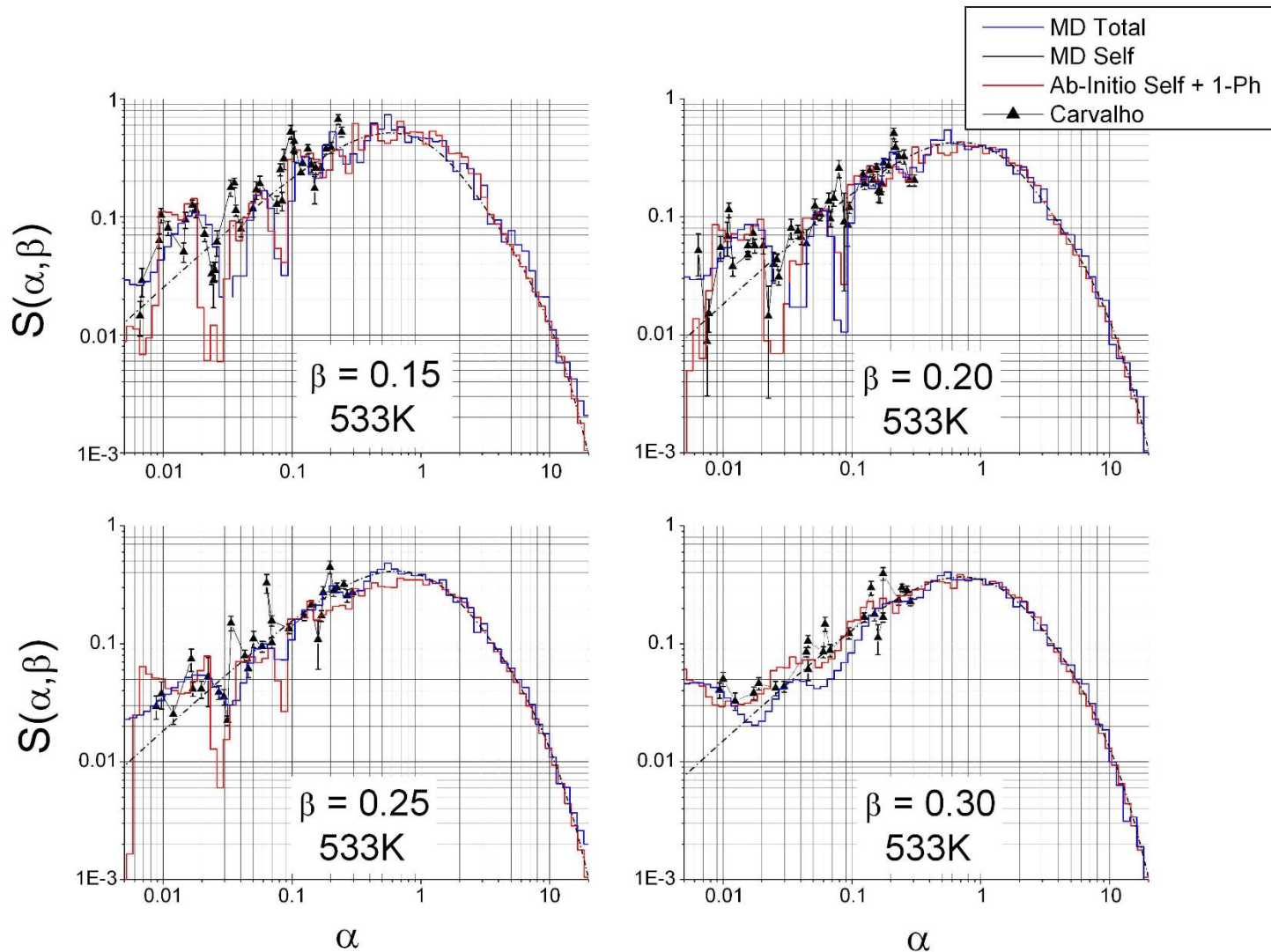
# Graphite



# Graphite



# Graphite Scattering Law

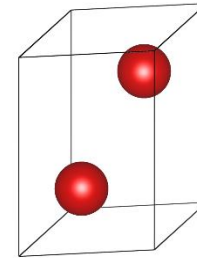


# Holistic Approach

## Ab Initio Molecular Dynamics

### □ Be – HCP ( $P6_3/mmc$ )

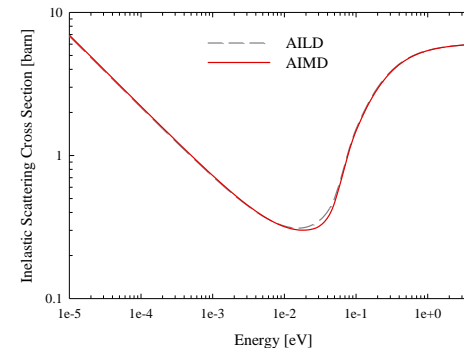
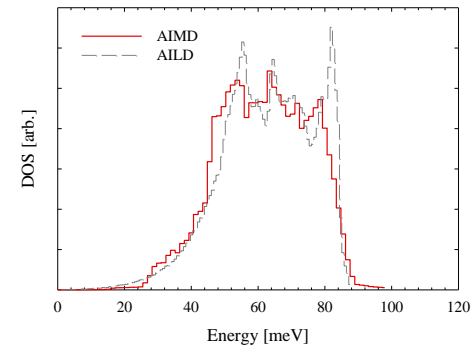
- $a=2.2856$  (2.27 AIMD)
- $c=3.5832$  (3.55 AIMD)



Accepted  
PHYSOR 2016

### □ VASP - AIMD

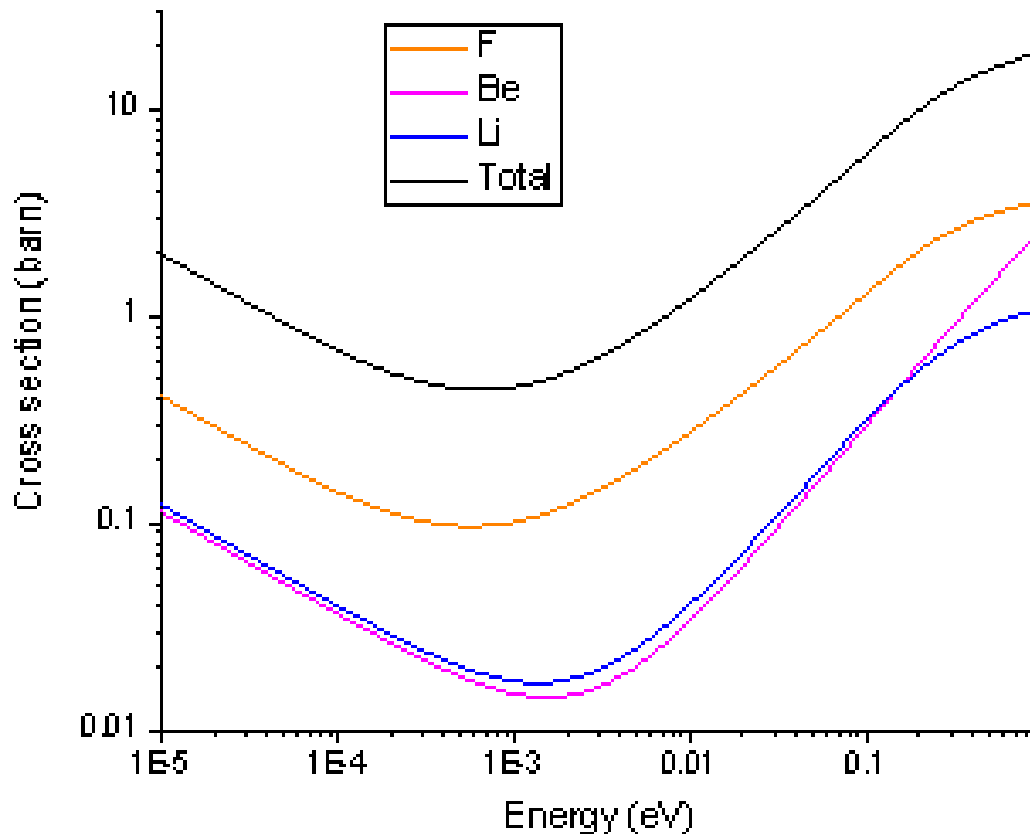
- 5x5x5 supercell
- GGA-PAW
- 3x3x3 k-mesh
- Fermi 'smearing' of electronic structure occupation
- 350eV Plane-wave cut-off
- 300 K under NVE conditions



# First Look

## Liquid FLiBe

- MD simulations combined with analysis to extract of the TSL data



Accepted  
PHYSOR 2016

# Summary

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- Developed a modern approach for thermal neutron cross section calculations based on the use of atomistic simulations
  - Ab initio lattice dynamics
  - Molecular dynamics (ab initio and classical)
  
- The approach is predictive
  - New materials
  - All states of matter (solid, liquid, gas)
  - Imperfect structure
  
- Coupling modern computations with targeted validation experiments should address all TSL data needs