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LABORATORY DIRECTED RESEARCH & DEVELOPMENT





# Getting sensitivities for various integral responses supporting nuclear data validation and adjustment

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Tools for sensitivities of different responses

Use cases

11/14/21

## EUCLID uses and developed tools to get sensitivities for

various different me	easurement
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	Observable					
Measurement Method	σ	v	β	PFNS		
Critical experiments	<ul> <li>✓</li> </ul>	✓		✓		
Neutron Multiplication Measurements	✓	✓	✓			
Reaction rate ratios	✓	✓		~		
Pulsed Spheres	✓					
Gamma/Neutron Leakage Spectra	✓			✓		
Delayed Neutron Measurements			✓			
Rossi-a	✓	✓	✓			
Reactivity Coefficient	<ul> <li>✓</li> </ul>		$\checkmark$			



Different measurement types give complimentary data which can help to disentangle compensating errors and improve overall validation of nuclear data.



Tools for sensitivities of different responses

Use cases

## Using existing sensitivity tools to obtain a diverse collection of sensitivity profiles

- Brute-force sensitivities with FRENDY/SANDY
  - Direct modification of ENDF/ACE files —
  - Requires individual isotope-reaction-energy perturbations and independent MCNP calculations
  - O. Cabellos and A.R. Clark computed pulsed- sphere sensitivities with this methodology\*

\*D.Neudecker, ANE 159, 108345, (2021).

### SENSMG

- Tool wrapped around the LANL deterministic transport code PARTISN
- Requires using multi-group cross sections
- Computed reaction rates, spectral indices (J. Alwin), and subcritical multiplication (A.R. Clark)  $\rightarrow$ sensitivities

### Tools for sensitivities of different responses





#### **Fission Spectrum**



Use cases

## Deriving and computing more sensitivities through existing capabilities

Using k<sub>eff</sub> sensitivities, T. Cutler, J. Hutchinson, N. Kleedtke have derived and simulated other response sensitivities

Beta-effective

 $\beta = 1 - \frac{k_p}{k}$ 

• Reactivity coefficients

$$\Delta \rho_{a-b} = \left(1 - \frac{1}{k^{(a)}}\right) - \left(1 - \frac{1}{k^{(b)}}\right)$$

Rossi-alpha

$$\alpha = \frac{k_p - 1}{l}$$



Tools for sensitivities

Different responses

Use cases

# M. Rising developed new fixed-source sensitivity in MCNP for easier execution and faster run-time.

- Central difference, brute force calculations (FRENDY/SANDY codes)
  - Difficult to distinguish Monte Carlo statistics from small perturbations
  - Computationally expensive
- Differential operator (PERT card)
  - More complicated to use (many input cards needed = isotope x reaction x energy)

689

692

690 pert101:n method=2 cell=1 2

rho=-3.0358e+01

Use cases

- Limited to reactions (no multiplicity, spectrum or angular distributions)
- Adjoint-weighting (new FSEN card) ———
  - Easy to use with accurate results (like KSEN)
  - Reactions, multiplicity, and energy / angular distributions all available





mat=9999

erg=1e-11 2.96937e-09

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rxn=2

# Verification of the new sensitivity methodology has been undertaken.

- Performing extensive verification to provide a robust and accurate capability
  - Comparison between FSEN and PERT



- Comparison between FSEN and central difference
  - Verification of FSEN using FRENDY results -



#### LLNL Pulsed Sphere of 0.7 MFP Pu



## Computational benefits of the new MCNP6 sensitivity methodology are promising

- FRENDY/SANDY vs. FSEN efficiency
  - Figure of merit (FOM)
    - Measure of Monte Carlo calculational efficiency
    - $FOM = \frac{1}{\text{Time}(CPU) \cdot \text{Var}(S)}$
    - Average efficiency gain for fission reaction
      - $7.3 \cdot 10^5 \text{ x}$
- PERT vs. FSEN efficiency
  - Computational cost is roughly equivalent
  - Pre- and post-processing of PERT results not automated ·
  - No optimization of the prototype FSEN capability (yet)
    - Tools for sensitivities of different responses



Use cases

Human cost to use PERT may be higher with how <u>easy</u> it is to use the new FSEN capability

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# W. Haeck programmed FAUST tool that processes all different output formats into common json format.

- FAUST
  - Input / output processing of MCNP, SENSMG results
  - Handling of all sensitivity data for use in nuclear data adjustment and experiment design optimization
  - Common and easy to use JSON format support (new MCNP6 FSEN output also available in JSON)
  - Crater tool used to **rapidly** study impact of nuclear data changes on validation benchmarks \_\_\_\_\_\_

## Tools for sensitivities of different responses

#### **Output from Crater Tool within FAUST**

	<b>====</b> Modifie	Pu239 d react	==== ion(s)						
esults	Benchma	rk Name			Diff.		Old Bias		New Bias
r data	MIX-COM MIX-COM MIX-MET MIX-MET	P-FAST- P-FAST- -INTER- -MIXED-	002-00: 003-00: 004-00: 001-00:	1   1   1   1	(pcm) -27 -28 -18 -3		(pcm) -494 -73 426 81		(pcm) -521 -101 408 78
t	PU-MET- PU-MET- PU-MET- PU-MET-	FAST-00 FAST-00 FAST-00 FAST-00	1-001-9 2-001 5-001 6-001		-15 -14 -20 -22		44 133 -73 -30		29 119 -93 -52
lear	PU-MET- PU-MET- PU-MET- PU-MET- PU-MET-	FAST-00 FAST-01 FAST-01 FAST-01 FAST-01	8-001 1-001 2-001 3-001 5-001		-20 -13 -24 -23 -20		-239 77 106 -559 -39		-259 64 82 -582 -59
	PU-MET- PU-MET-	FAST-04 FAST-04	5-006- 5-007- 02-001		-19 -19 -20		723 691		704 672 1676
	PU-MET- PU-MET-	INTER-Ø	03-001 04-001		-19 -19		154 -71		135 -90
Use cases	Manuall Modifie 90 expe St. Dev Average	Pu239 y set cl d react riments iation: Bias :	<pre>anges ion: with old 495 191</pre>	['tota absolu N => 4 => 1	l nu'] te cha ew 90 pcm 76 pcm	0. Inge	999 68 > 0.0	-50 pc	0 keV m

## **EUCLID** Uses sensitivities for many use-cases.

Neutron Transport Simulation (MCNP)

**Validation Experiments** 



### ML-Augmented Search for Compensating Errors



**Experiment Refines Nuclear** Data to Improve Simulations

> ML-Optimized Experiment to Resolve Compensating Errors



Tools for sensitivities of different response

Use cases

### One use-case is adjustment: we can filter to and select from our diverse set of benchmarks for adjustment

Original ○ Inflate Uncertainty for Labeled Questionable Benchmarks Adjustment tool developed by Mike Grosskopf. ● No Correction ○ Pulsed Sphere GP Correction for Long TOF Set Gaussian Process TOF Correlation Length (ns): 10.0 25.0 50.0 100.0 150.0 200.0 Set TOF Gaussian Process Standard Deviation (cm^-2 ns^-1): 1.0e-5 1.0e-4 1.0e-3 1.0e-2

#### Select Benchmarks to Adjust To:

s	ELECT ALL	DESELECT ALL									
4	Benchmark Type	es 🌣 Experi	ment Names 🍦	Experimental Value	Calculated Value	Expt. Uncertainty	Calc. Uncertainty	Total Uncertainty	Bias (sigmas)	Time-of-Flight \$	D-Optimality
	filter data		PU-MET								
	criticalit	y PU-MET-FAST	-001-001-s	1	1.00044	0.00129	0.0008	0.001292478239662	-0.3404312633650697		0.00012225205129393036
	criticalit	y PU-MET-FA	ST-002-001	1	1.00133	0.002	0.0008	0.0020015993605114	-0.664468637550046		0.0000826247873474197
	criticalit	y PU-MET-FA	ST-005-001	1	0.99927	0.0013	0.00009	0.0013031116606031	0.5601975809672006		0.00010273708220366292
	criticalit	PU-MET-FA	ST-006-001	1	0.9997	0.003	0.0001	0.0030016662039607	0.09994449069790533		0.0001225526093910824
	criticalit	PU-MET-FA	ST-008-001	1	0.99761	0.0006	0.00009	0.0006067124524847	3.939263138925394		0.00010079873258719962
	criticalit	PU-MET-FA	ST-011-001	1	1.00077	0.001	0.00011	0.0010060318086422	-0.7653833540702604		0.00009320550584870178
	criticalit	PU-MET-FA	ST-012-001	1.0009	1.00196	0.0021	0.0001	0.0021023796041628	-0.504190583803808		0.00010864902908367885
	criticalit	PU-MET-FA	ST-013-001	1.0034	0.99781	0.0023	0.00009	0.0023017601960239	2.428576186892256		0.00009735494910760852
	criticalit	PU-MET-FA	ST-015-001	1.0041	1.00371	0.0026	0.0001	0.0026019223662515	0.1498891761946572		0.00009873491029936234
	criticalit	PU-MET-FA	ST-016-001	0.9974	1.0206	0.0042	0.00012	0.0042017139360027	-5.5215562871163275		0.00006938027010215084
	criticalit	PU-MET-FA	ST-016-002	1	1.01019	0.0038	0.00012	0.0038018942647054	-2.680242871191348		0.0000687562355783532
	criticalit	PU-MET-FA	ST-016-003	1	1.0085	0.0033	0.00011	0.0033018328243567	-2.5743277907039457		0.0000686450512013453
	criticalit	PU-MET-FA	ST-016-004	1	1.00787	0.003	0.00012	0.0030023990407672	-2.6212371817135374		0.00006887348720853011
	criticalit	y PU-MET-FA	ST-016-005	1	1.00776	0.0034	0.00012	0.0034021169879943	-2.280932733172958		0.00006858819619233457
_			am 03.6 00.6	,	1 01003	0 0033	0.00011	0 0000010000000000	3 133534707136657		A AAAACAAAACCC3E1034A4



1.0e-1

400.0

1.0e00

### Adjusting to Jezebel we can explore the impact on nuclear data and the propagated prediction and uncertainties for Jezebel

### **EUCLID** Adjustment Visualization

Nuclear Data Adjustment to Benchmark Data by Augmented GLLS



UPDATE



Tools for sensitivities of different responses

Use cases

## One warning (among many): adjusting to data with unaccounted for biases leads to unreasonable results.

#### Select Benchmarks to Adjust To:

SELECT ALL	DESELECT ALL							
Benchmark Types	Experiment Names \$\overline\$	Experimental Value	Calculated Value	Expt. Uncertainty	Calc. Uncertainty	Total Uncertainty	Bias (sigmas)	Time-of-Flight
AA pulse	pu pu							
pulsed sphere	pu0.7b_020	0.0007147913970509	0.000681932	0.0000359215	0.000005932808399999999	0.00003640813614786001	0.9025289544472154	225
pulsed sphere	pu0.7b_021	0.0007122360609277	0.000682894	0.0000358595	0.000060777566	0.00003637090685615144	0.8067453760157601	227
pulsed sphere	pu0.7b_022	0.0007600208464308	0.000692146	0.000037023	0.000063677432	0.00003756661659587068	1.806786252831212	229
pulsed sphere	pu0.7b_023	0.0008498409111599	0.0007241289999999	0.0000391185	0.0000072412899999999999	0.00003978307835140589	3.1599342325795003	231
pulsed sphere	pu0.7b_024	0.0008191129942789	0.000767262	0.00003841550000000005	0.000066751794	0.00003899113566276551	1.3298149283816565	233
pulsed sphere	pu0.7b_025	0.0007823800375085	0.000814031	0.0000375555	0.000006837860400000006	0.00003817292122827238	-0.8291469835967908	235
pulsed sphere	pu0.7b_026	0.0009983059399155	0.000855741	0.000042355500000000005	0.0000071882244	0.00004296113301898305	3.318463222385344	237
pulsed sphere	pu0.7b_027	0.0010256480364333	0.000907246	0.0000429255	0.0000075301418	0.00004358097733849147	2.716828388534677	239
pulsed sphere	pu0.7b_028	0.0010420660710246	0.000941066	0.000043263	0.00000781084780000001	0.00004396244434008151	2.2974170918088843	241
pulsed sphere	pu0.7b_029	0.0010045026300141	0.000975567	0.000042485500000000005	0.000007999649400000001	0.0000432320725940004	0.6693093409108383	243
pulsed sphere	pu0.7b_030	0.0009608063823081	0.00102875	0.000041562	0.00008332875	0.00004238911003743326	-1.6028554888720192	245
pulsed sphere	pu0.7b_031	0.0011343775884738	0.00109541	0.00004511850000000005	0.000008872821	0.00004598267059173533	0.8474407417476941	247
pulsed sphere	pu0.7b_032	0.0011301612838705	0.00115419	0.0000450355	0.00000888726300000001	0.00004590402709873252	-0.5234555146505547	249
pulsed sphere	pu0.7b_033	0.0012248364872336	0.00122588	0.000046865	0.00009316688	0.000047782098115186864	-0.02183899007290044	251
pulsed sphere	pu0.7b_034	0.0012535840186192	0.00125507	0.000047407	0.00009538532	0.00004835708057477233	-0.030729344351182287	253
pulsed sphere	pu0.7b_035	0.0012555005207115	0.0013207	0.0000474425	0.00000964111	0.0000484122072238201	-1.3467570067001615	255
pulsed sphere	pu0.7b_036	0.0013698518122229	0.00136453	0.0000495365	0.000009961069	0.00005052808850404655	0.10532383829389909	257
pulsed sphere	pu0.7b_037	0.001306032292547	0.00141788	0.000048379	0.00000992516	0.00004938660184731888	-2.264737869570025	259
pulsed sphere	pu0.7b_038	0.0014092678719227	0.00151701	0.00005024	0.000011377575	0.00005151220062160638	-2.091584649406505	261
pulsed sphere	pu0.7b_039	0.0015691041464263	0.00156268	0.00005299	0.00010782492	0.00005407589327722719	0.1187987111625835	263



## As mentioned however, adjusting to data with unaccounted for biases leads to unreasonable results

### **EUCLID** Adjustment Visualization

Nuclear Data Adjustment to Benchmark Data by Augmented GLLS





Tools for sensitivities of different responses

Use cases

## Adding in a Gaussian process with tunable parameters to account for the bias uncertainty for long TOF mitigates this issue:

**EUCLID** Adjustment Visualization

Nuclear Data Adjustment to Benchmark Data by Augmented GLLS





Tools for sensitivities of different responses

Use cases



### Experiments Underpinned by Computational Learning for Simulations Improvements in nuclear

<u>D</u>ata







Wim Haeck



Michal Herman





Denise Neudecker



Jennifer Alwin



Alexander Clark



Juliann Lamproe

**Robert Little** 



**Michael Rising** 





Experiments



Michael Grosskopf



Noah Kleedtke



Isaac Michaud



**Travis Smith** 



Scott Vander

Wiel





**Theresa Cutler** 

Jesson Hutchinson



### Summary

- EUCLID computed sensitivities of several integral responses to nuclear data,
- EUCLID leveraged existing tools, but new ones were also developed,
- EUCLID uses sensitivities for validation, finding where compensating errors could hide in our libraries, optimization of integral experiments and adjustment,
- We are happy to share our sensitivities after release and are establishing contact with the NEA about possible sharing these data through their tools to the community.





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### Backup



## **Critical Experiments (k<sub>eff</sub> inference)**

- At or very near delayed critical
- Convenient; uncertainties very small (~0.1%)
- Well understood
- Requires little inference
- Most common type of measurement  $\rho(\$) = \frac{\rho}{\beta_{eff}} = \frac{(k_{eff}-1)}{(k_{eff}\beta_{eff})} = \Lambda\omega + \sum_{i=1}^{\circ} \frac{\beta_i \omega}{\omega + \lambda_i}$  in validation
- Sensitivity Simulation capabilities
  - KSEN (already developed and tested)
  - Whisper 1.1
  - Faust for analysis
    - Critical experiments are the traditional metric for integral experiment response in nuclear data evaluation.





### **Neutron noise experiments**

- Description
  - Subcritical measurements
  - Greater Sensitivities to spontaneous fission nuclides

 $R_1 = \epsilon M_L v_{S1} F_S$ 

$$R_2 = \epsilon^2 M_L^2 F_S \left( \bar{v_{52}} + \frac{M_L - 1}{\bar{v_{I1}} - 1} \bar{v_{51}} \bar{v_{I2}} \right)$$



FUND-NCERC-PU-HE3-MULT-001-007\_Pu-239\_cross section\_nubar



## Sensitivity Simulation capabilities

- SENSMG
- Faust for analysis





Neutron noise measurements have higher sensitivities to nuclides with significant spontaneous fission emission versus keff.

## **Reaction Rate Ratios**

- Description
  - comparison of reactions in two fissionable components
    - e.g. <sup>239</sup>Pu(n,f)/ <sup>235</sup>U(n,f)



$$\frac{\langle \sigma^x \rangle}{\langle \sigma^y \rangle} = \frac{\int \sigma^x(E) \,\psi(E) dE}{\int \sigma^y(E) \,\psi(E) dE}$$

Sensitivity Simulation capabilities SENSMG New FSEN card in MCNP6 (developed under this project) New portion of Faust for analysis



U238f/U235f Rxn Rate



Reaction rate ratios are insensitive to the value of keff

## **Pulsed Spheres**

- Description
  - Decouple from fissionable isotope
  - Focus on nuclear data above 5-15 MeV
    - Dependent on thickness of material and level structure
  - High sensitivity to angular distributions and PFNS







Pulsed spheres decouple sensitivity of fissionable and nonfissionable material, and significantly extend the energy range for validation.

## **Gamma/ Neutron Leakage Spectra**

- Description
  - Measure of the energy spectrum of the exterior of the assembly
  - Focus on gamma/ neutron energy measurements
  - Preferably subcritical system to not overwhelm detectors
  - Significant sensitivity to room return
- Sensitivity Simulation Capability
  - SENSMG
  - FSEN in MCNP6





Leakage neutron spectra measurements focus on the energy of the neutrons, independent of keff

## **Delayed Neutron Measurements**

- Description
  - Inference of the delayed neutron fraction and groups
  - Converts measured system values (e.g. neutron e-folding time, neutron lifetime) to calculated system values (keff)
  - Determines how reliant the system is on neutrons released later in time, which makes it controllable

$$\beta_{eff} = 1 - \frac{k_p}{k_{eff}}$$

- Sensitivity Simulation Capabilities
  - Currently requires two separate simulations
    - Faust for analysis (written under this project)

#### <sup>235</sup>U fission sensitivity



Delayed neutron sensitivities  $(S_{\beta eff,x})$ exhibit very different sensitives than  $k_{eff} (S_{k,x})$  for the same reaction.



Prompt and delayed neutrons have different energies, resulting in unique sensitivity profiles which are different from other responses such as keff.

## **Rossi-alpha Measurements**

- Description
  - Measurement at delayed critical and slightly subcritical
  - rate at which the prompt neutron population changes as a function of time
  - used to infer parameter neutron lifetime, I.
  - Can be used to infer reactivity



Sensitivity Simulation Capabilities



Rossi-alpha measurements probe the system lifetime, a key aspect of time dependent responses.

Eljen [3].

12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 20 31 32 33 34 35 36 37 38 39

 $-\rho$ 

MCNP with PERT Future: FSEN

### **Reactivity Coefficients**

- Compare worth of a sample to worth of void in the location in the assembly
- Focused on effects from a small change (often 5-20 g)
- Removes some systematic unc



$$\Delta \rho_{a-b} = \left(\frac{1}{keff^{b}}\right) - \left(\frac{1}{keff^{a}}\right) = (\rho_{a}[\$] - \rho_{b}[\$])\beta_{eff}$$





### Sensitivity Simulation Capabilities

- Currently requires two separate simulations
- Faust for analysis (written under this project)

Reactivity coefficients remove some systematic uncertainties and show distinctly different sensitivities to keff

## **Typical Uncertainties for the Various Measurement Methods**

Measurement Method	
	Typical Uncertainty (%)
Critical experiments	0.1-0.5
Noutron Multiplication Massuraments	R1: 1-3,
	R2 and Y: 2-9
Reaction rate ratios	0.9-2.0
Pulsed Spheres	5-10
Gamma/Neutron Loakago Sportra	3-20 (depending upon
	energy bin)
Delayed Neutron Measurements	8
Rossi-a	1-3
Reactivity Coefficient	7

Values based on ICSBEP and IRPhEP benchmarks

