



TSL Nuclear Fuel Evaluations and Capabilities at NC State University

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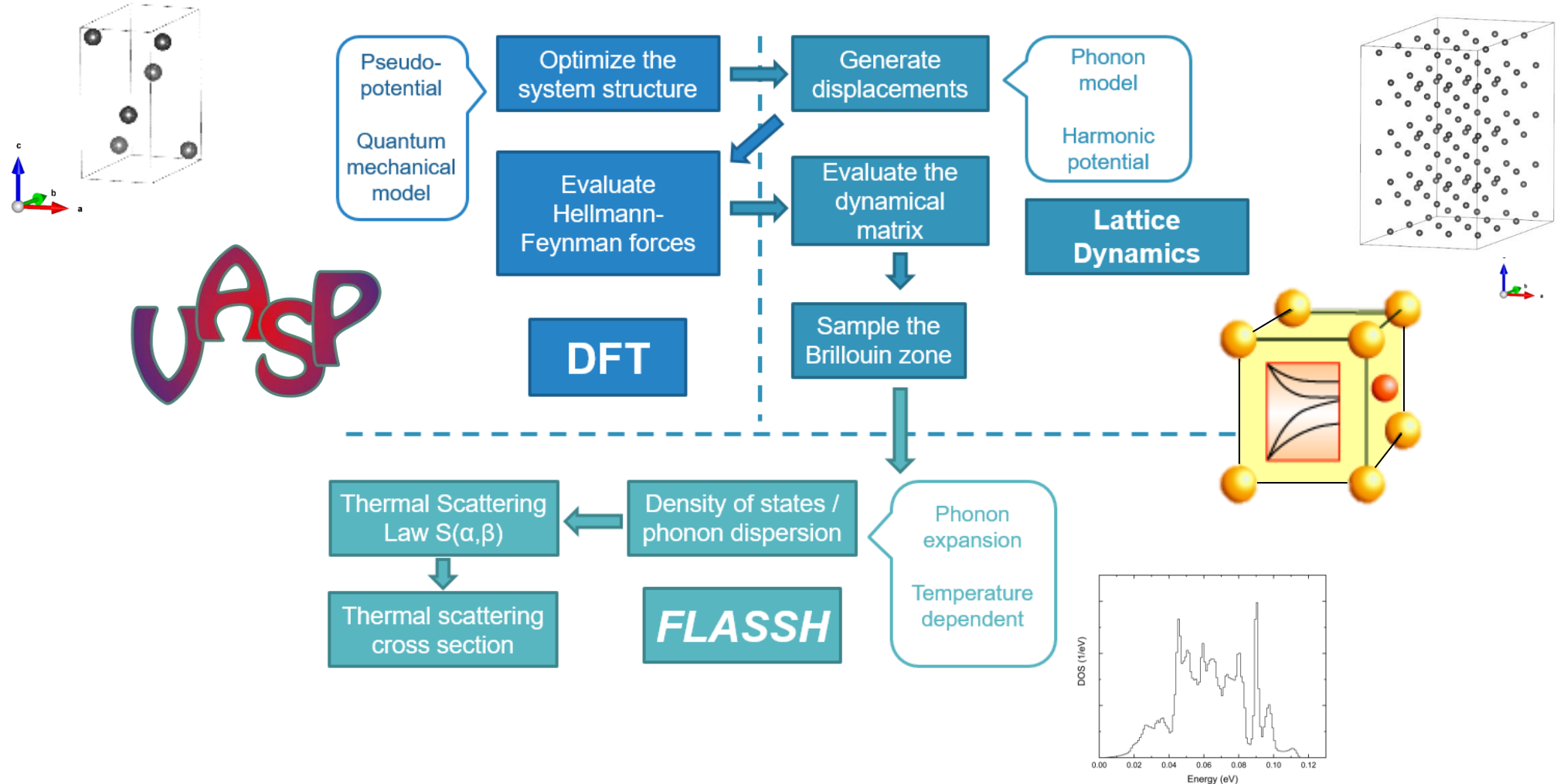
CSEWG 2022
November 1, 2022 BNL

Outline

- **Provide thermal scattering law (TSL) and cross section data to support advanced reactor modeling and criticality safety**
 - **Modeling and Simulations**
 - **Uranium Fuel Materials**
 - **Evaluation Updates**
 - **Benchmark Applications**



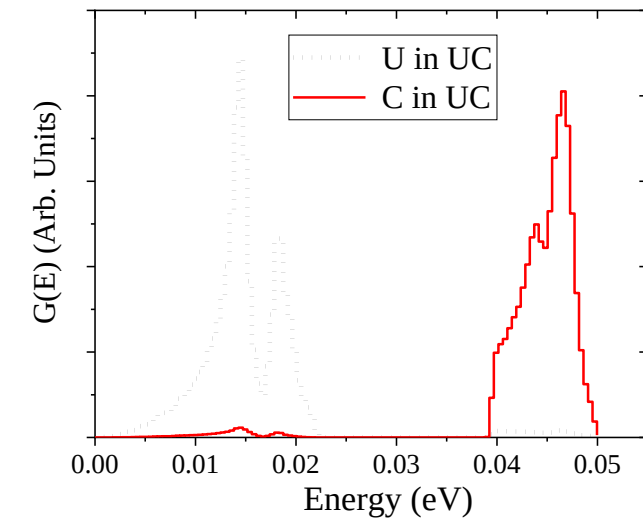
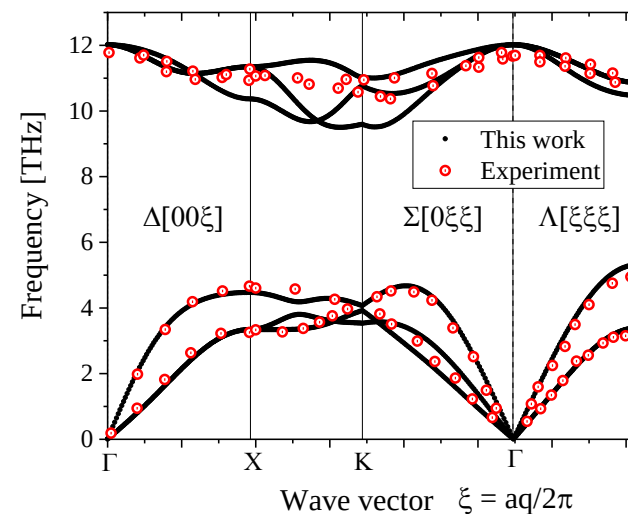
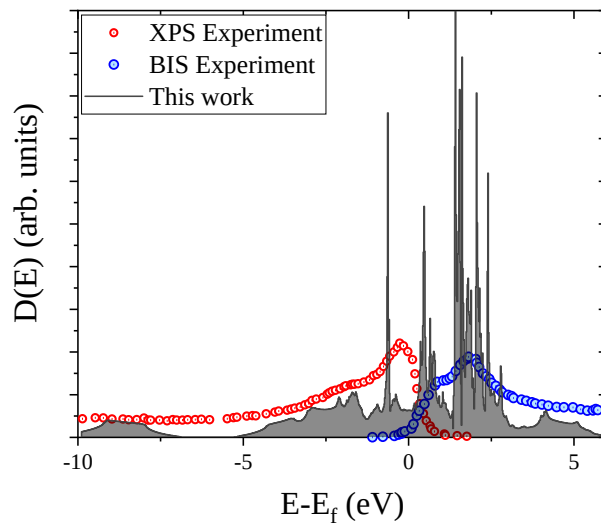
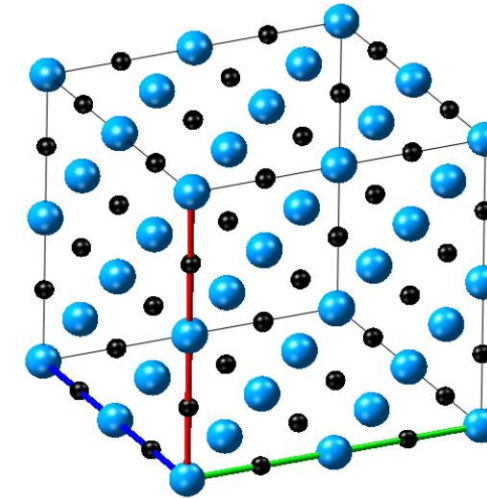
Material Modeling



Uranium Carbide

□ VASP DFT Framework

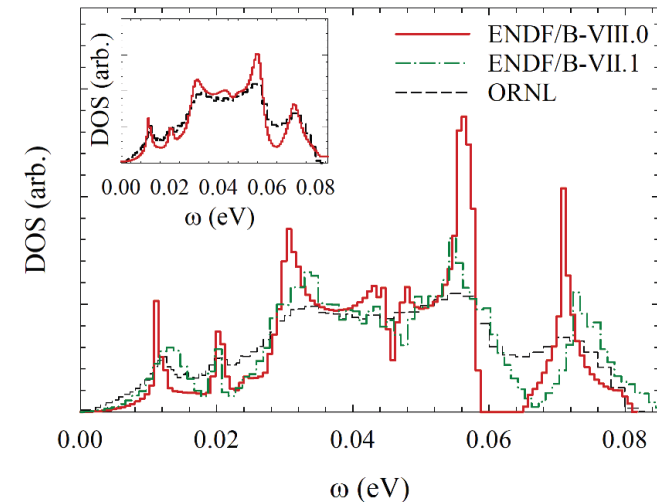
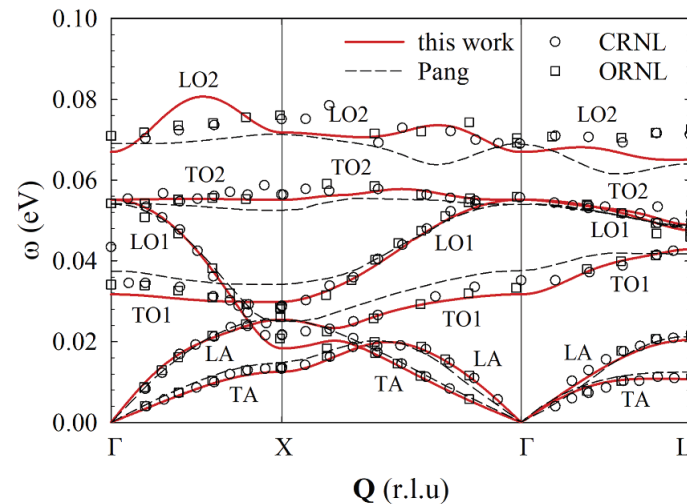
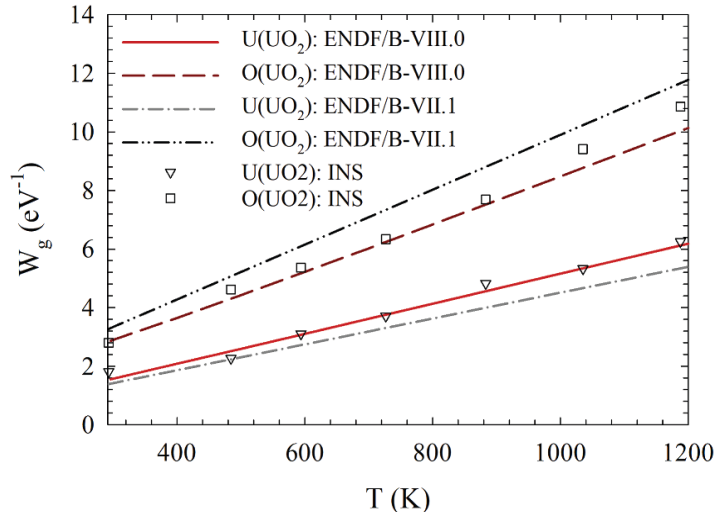
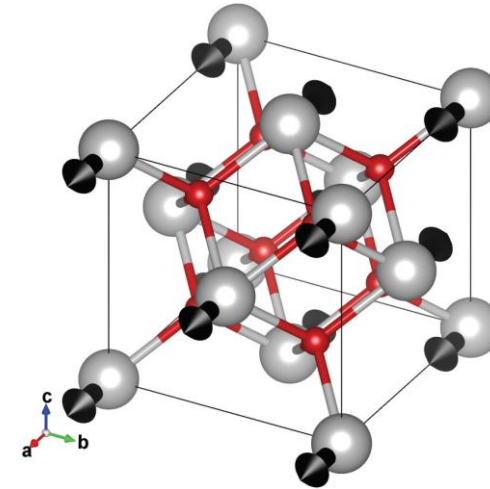
- GGA-PBESol Pseudopotential
- 1.9 eV Effective Hubbard Term
- Magnetic Structure Captured with Spin-Orbit-Coupling
 - AFM-I polarizations in x, y, z -> paramagnetic
 - 2x2x2 supercell (64 atoms)



Uranium Dioxide

□ VASP DFT Framework

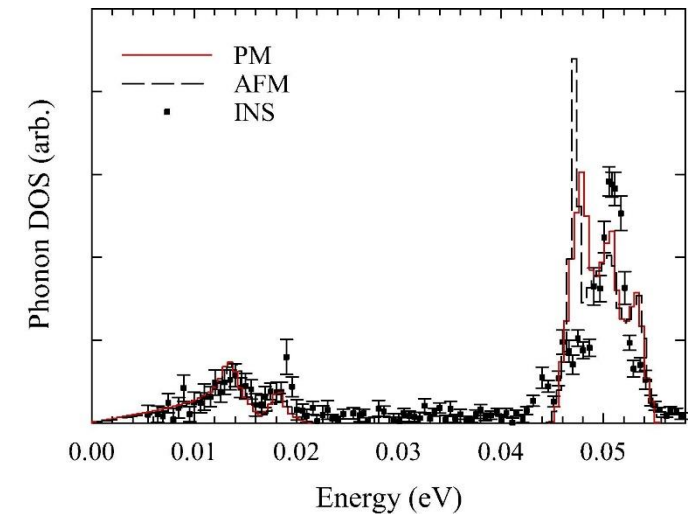
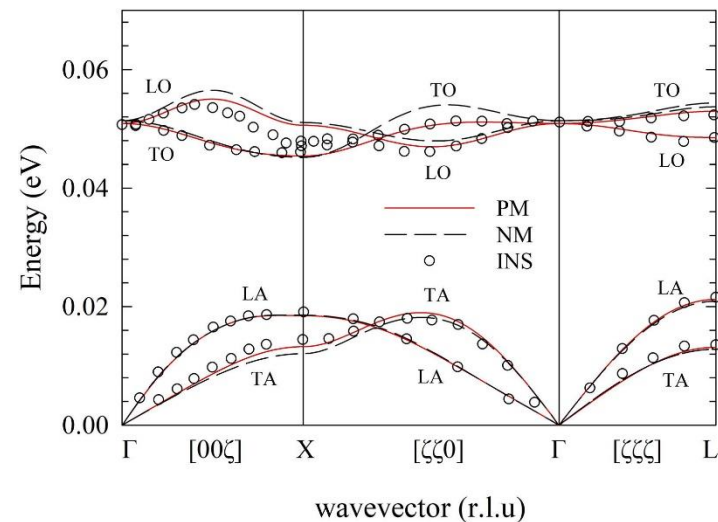
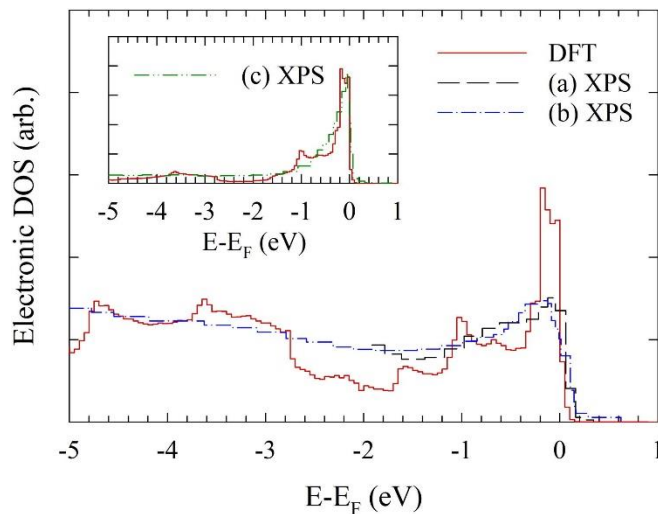
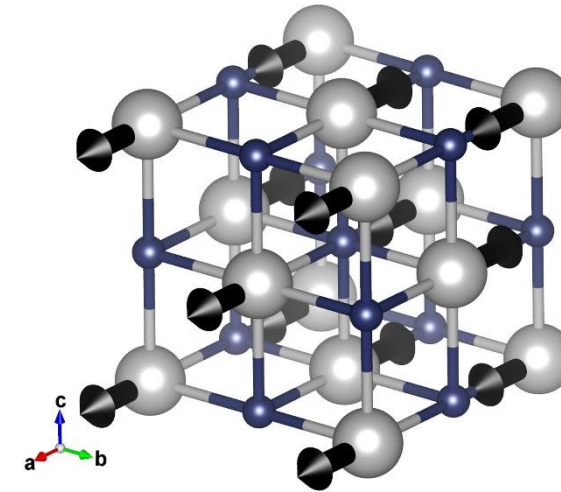
- GGA-PBE Pseudopotential
- 4.0 eV Effective Hubbard Term
- Magnetic Structure Captured with Spin-Orbit-Coupling
 - AFM polarizations in x, z $\rightarrow$ paramagnetic
 - 2x2x2 supercell (64 atoms)



Uranium Nitride

□ VASP DFT Framework

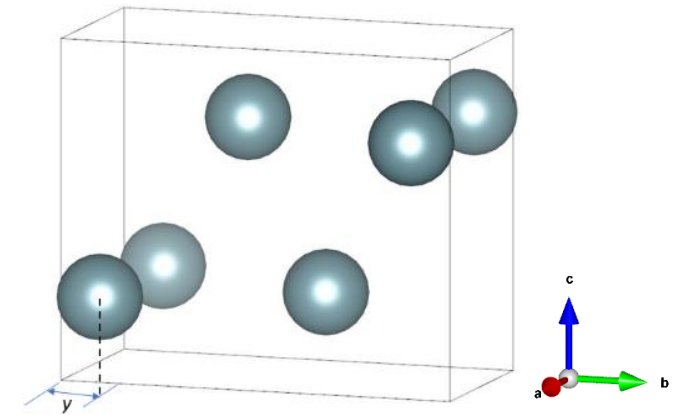
- GGA-PBE Pseudopotential
- No Hubbard Term
- Magnetic Structure Captured with Spin-Orbit-Coupling
 - AFM polarizations in x, y, z $\rightarrow$ paramagnetic
 - 2x2x2 supercell (64 atoms)



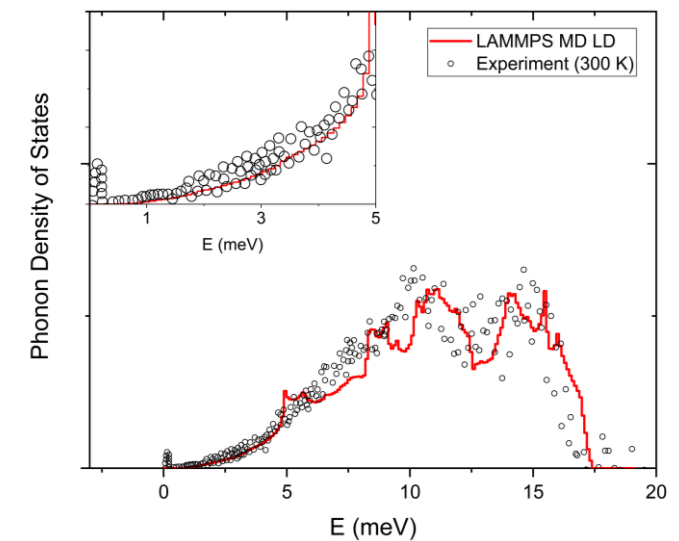
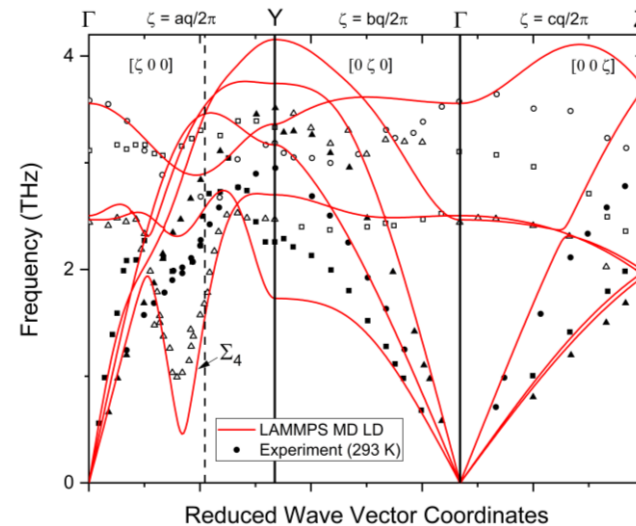
Uranium-Metal

❑ Molecular Dynamics Lattice Dynamics

- Temperature dependence
- Short times only
 - EAM potential
 - 20x10x10 supercell (20,000 atoms)
 - 0.04Å displacement



	B (GPa)	G (GPa)	σ (Å)
Experiment (50 K)	111	---	---
Experiment - 2	97.9	73.1	0.20
Experiment - 3	135.5	---	---
Experiment - 4	---	59-84	0.19-0.28
This Work (MD)	137	56	0.32

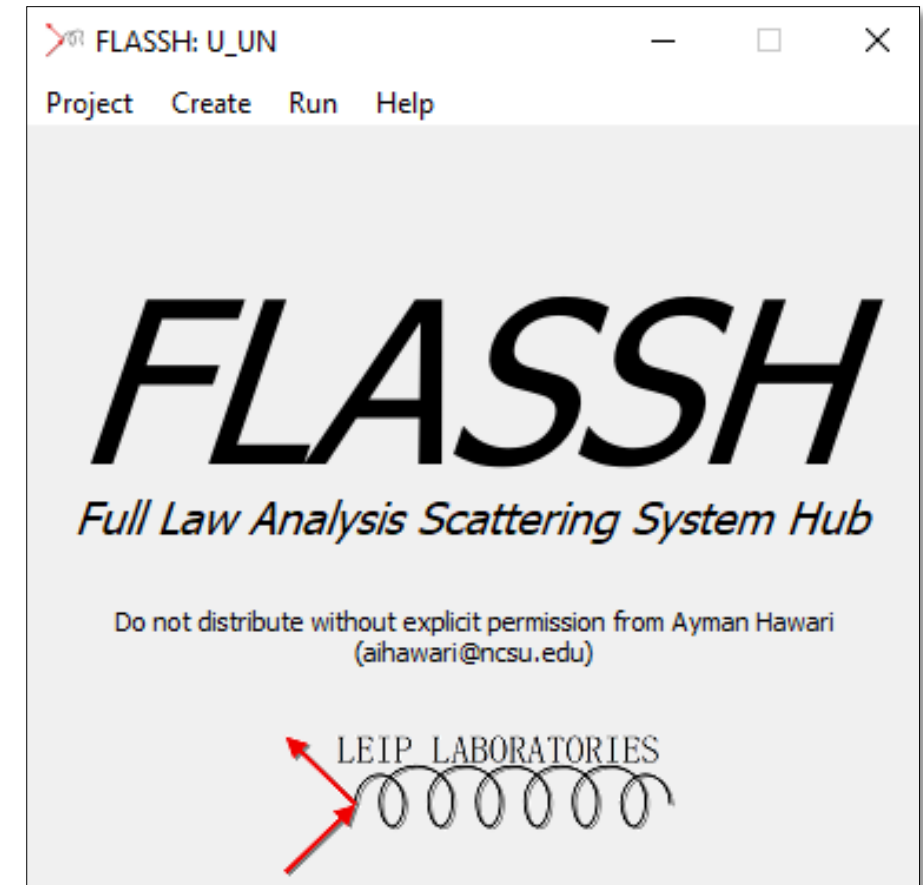


FLASSH

- **FLASSH: Full Law Analysis Scattering System Hub**
 - Modern thermal scattering evaluation code
 - Improves the physics of TSLs
 - Convenient user interface
 - Advanced physics

$$\frac{\partial^2 \sigma}{\partial \Omega \partial E'} = \frac{1}{4\pi} \sqrt{\frac{E'}{E}} [\sigma_{coh} S(\alpha, \beta) + \sigma_{inc} S_s(\alpha, \beta)]$$

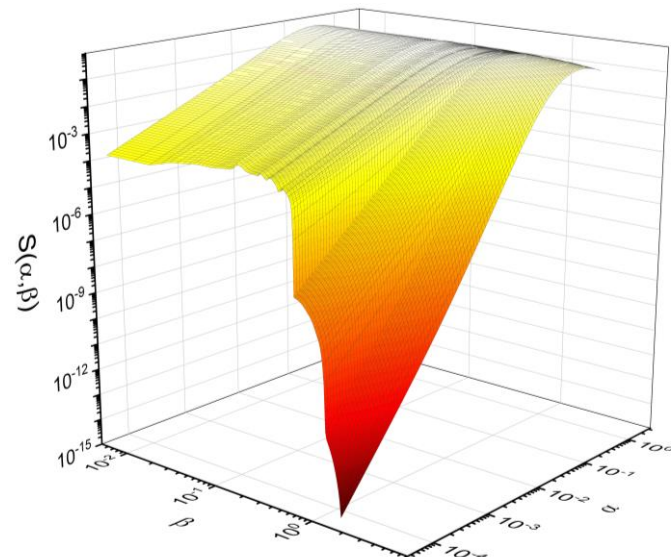
$$S(\alpha, \beta) = \frac{1}{k_B T} \frac{1}{2\pi \hbar} \int e^{i(\vec{k} \cdot \vec{r} - \omega t)} G(\vec{r}, t) d\vec{r} dt$$



TSL Evaluations

○ Updates to Existing Libraries

- Updated lattice parameters used to evaluate the coherent elastic
- Updated input free atom cross sections and masses to reflect the current ENDF/B-VIII.0 values
- Evaluations completed with *FLASSH*



```

1.840000+2 2.330248+2      -1      0      0      0 84 1451
0.000000+0 0.000000+0      0      0      0      6 84 1451
1.000000+0 5.000000+0      0      0     12      8 84 1451
0.000000+0 0.000000+0      0      0     31      3 84 1451
U(UO2)   LEIP LAB   EVAL-SEP22 N.C. Fleming, A.I. Hawari
                        DIST-
-----ENDF/B-VIII.0   MATERIAL   84
-----THERMAL NEUTRON SCATTERING DATA
-----ENDF-6 FORMAT

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```

Temperatures = 296 400 500 600 700 800 1000 1200 K
HISTORY
-----
This library was produced by the Low Energy Interaction Physics
(LEIP) group at NC State University, USA. The inelastic
scattering thermal scattering law data for UO2 were developed
using ab initio lattice dynamics methods [1,2]. The Full Law
Analysis Scattering System Hub (FLASSH) code was used to
produce File 7 MT=2 and 4 for U in UO2 and O in UO2 [3]. MAT=84
and ZA=184 were used for U in UO2 (HEU).

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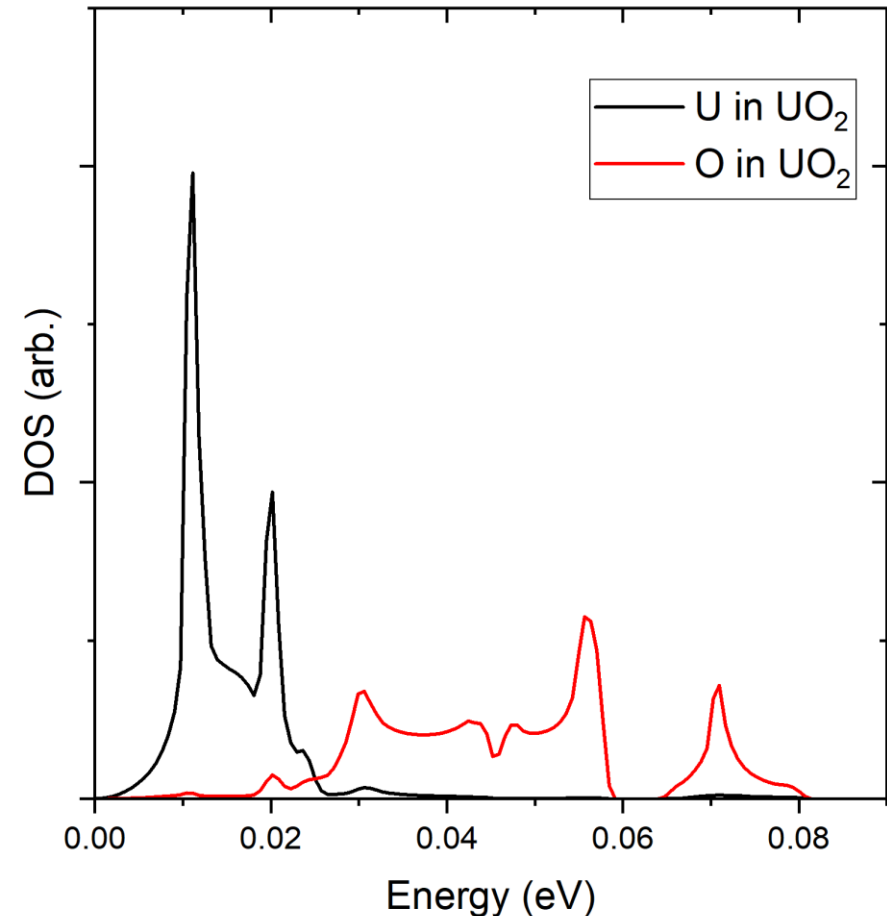
REFERENCES
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1. A.I. Hawari, "Modern Techniques in Inelastic Thermal Neutron
   Scattering Analysis," Nuclear Data Sheets 118 (2014) 172.
2. J.L. Wormald, A.I. Hawari, "Ab Initio Generation of Thermal
   Scattering Law for Uranium Dioxide," Trans. Am. Nucl. Soc.,
   115 (2016).
3. N. C. Fleming et al., "FLASSH 1.0: Full Law Analysis
   Scattering System Hub," Trans. Am. Nucl. Soc., 125 (2021).

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Enrichment Evaluations

► TSL Evaluations for Varying Enrichment

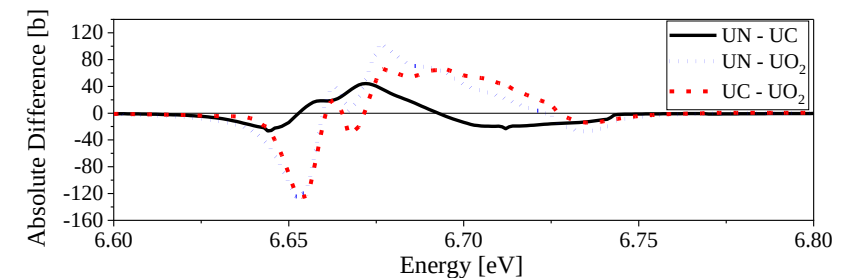
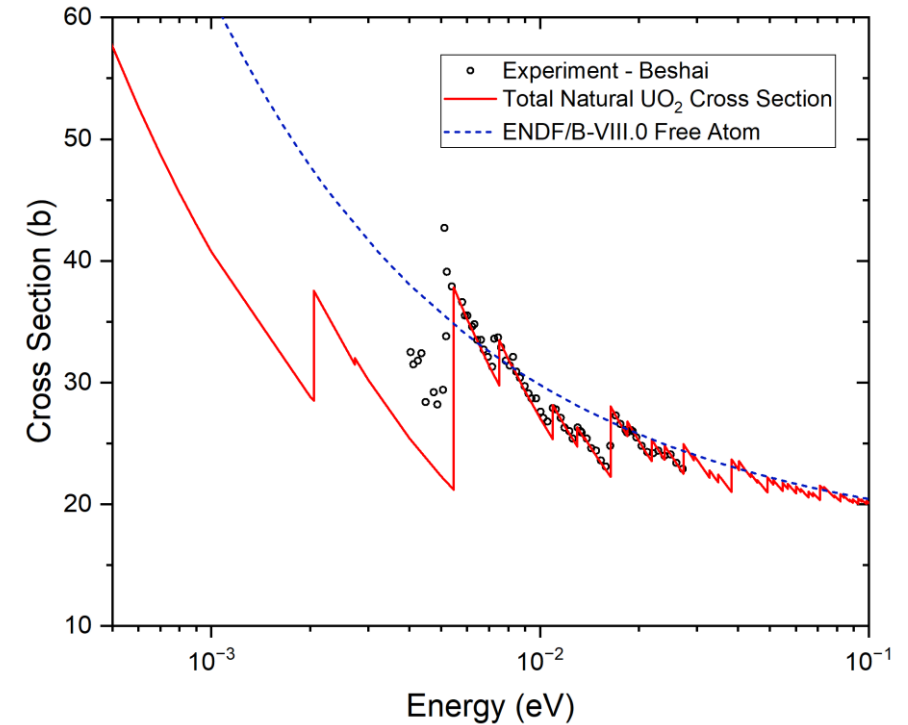
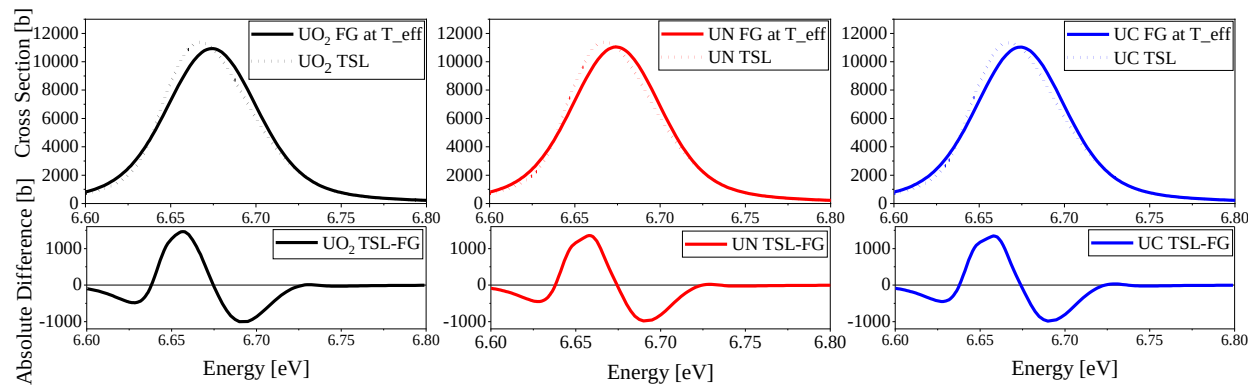
- Inputs Varied
 - **Mass**
 - **Free Atom Cross Section**
- DOS constant
- Natural (NIST natural elemental isotopic weighting)
- 5% U-235
- 10% U-235
- HEU (100% U-235)



Uranium Cross Sections

○ Benchmarks

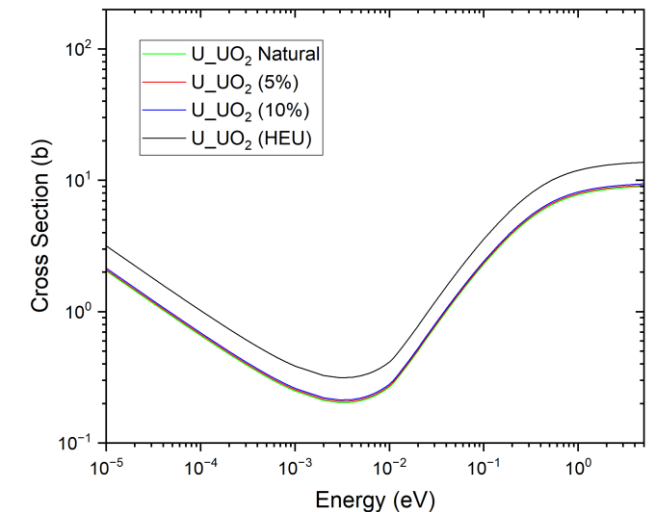
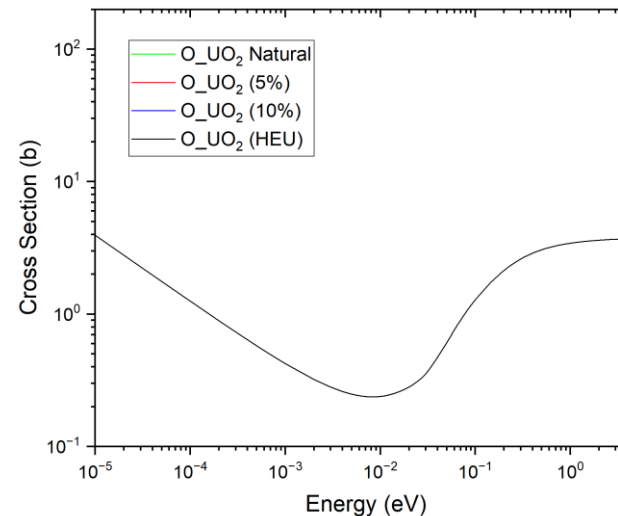
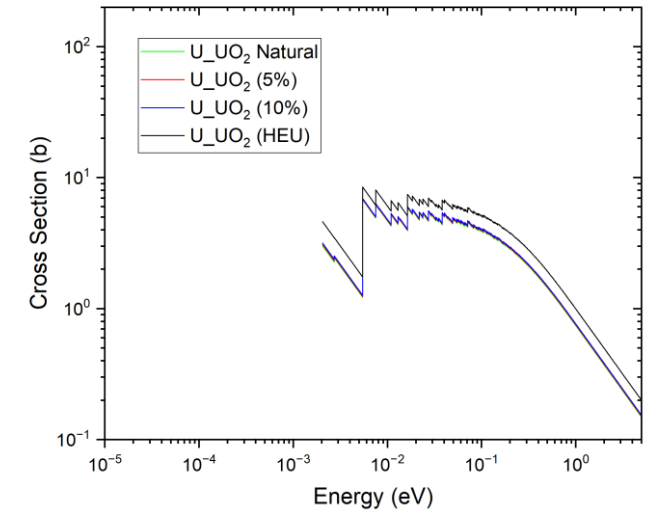
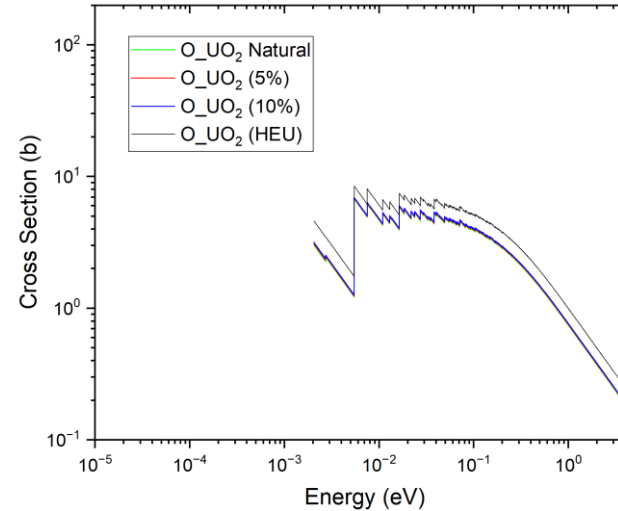
- Total Cross Section Measurements
- Doppler Broadened Resonances
- Natural Cross Sections



Uranium Dioxide

○ UO_2 TSLs

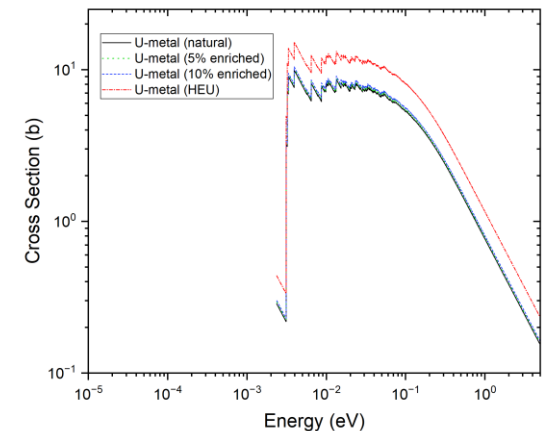
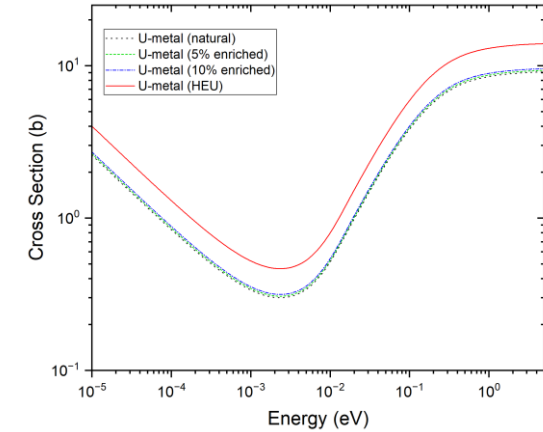
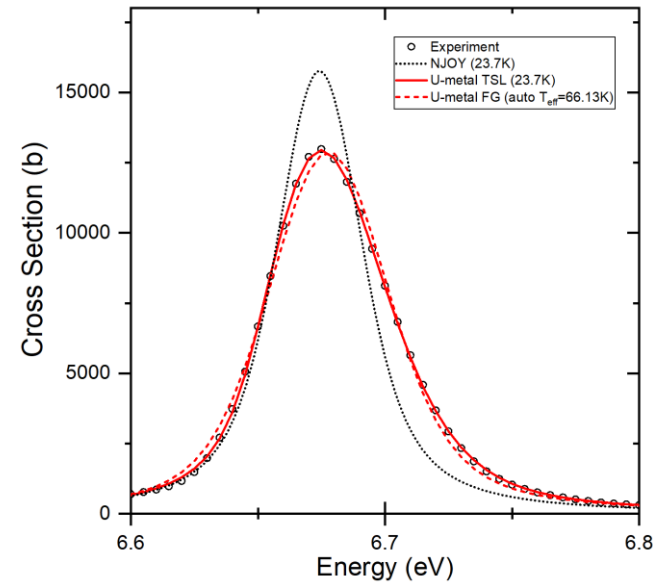
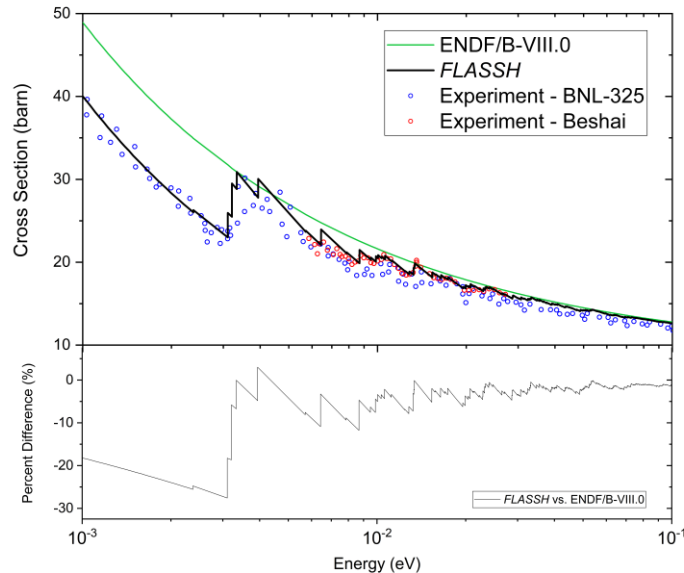
- Coherent elastic reflection of compound demonstrates effects from the varies cross section and mass impacts of the uranium weighting
- Inelastic cross sections are a reflection of the unchanged DOS



Uranium-Metal

○ U-metal TSLs

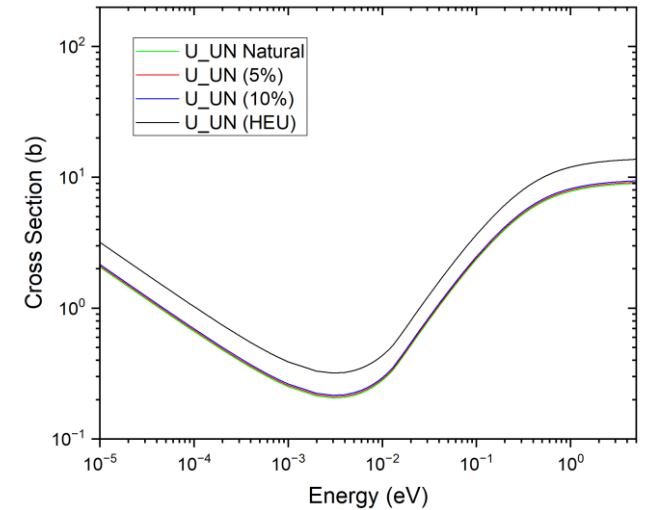
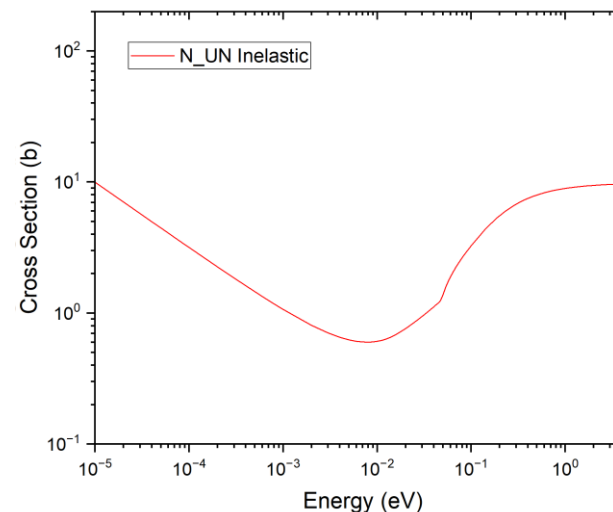
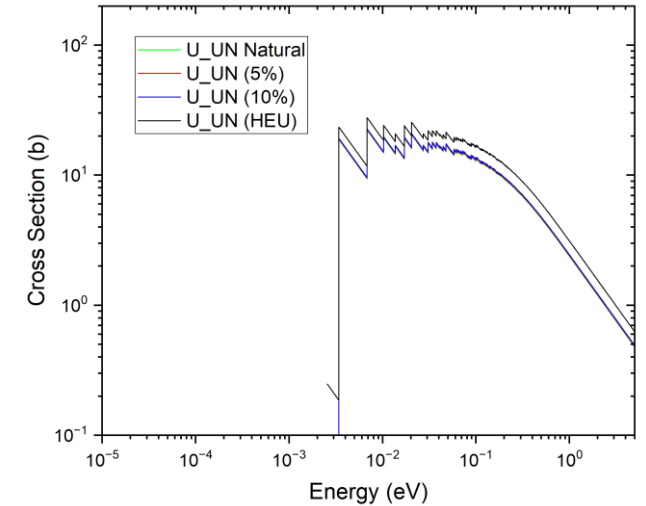
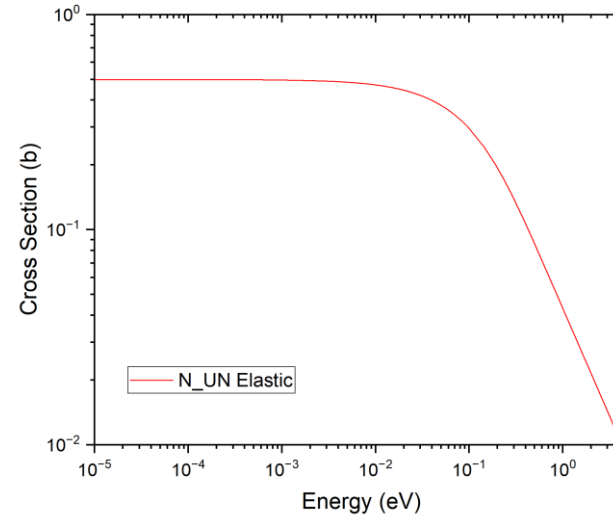
- Benchmarked against total cross section measurements
- Demonstrated accurate structure in ability to capture the correct structure of U-metal Doppler broadened resonance structure
- Enrichment-dependent libraries available



Uranium Nitride

○ UN TSLs

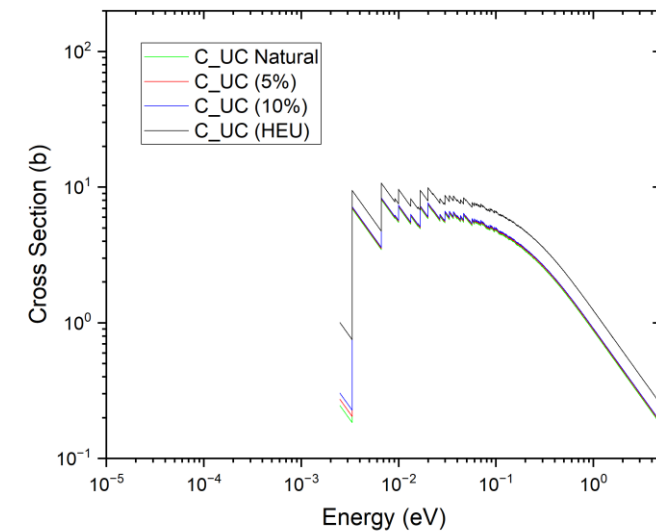
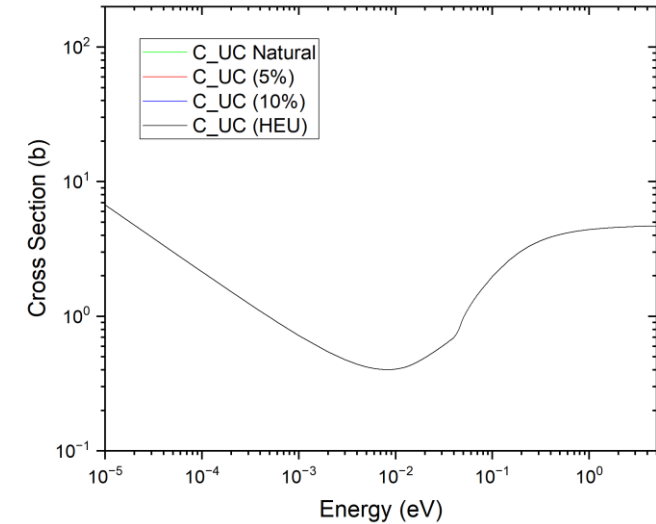
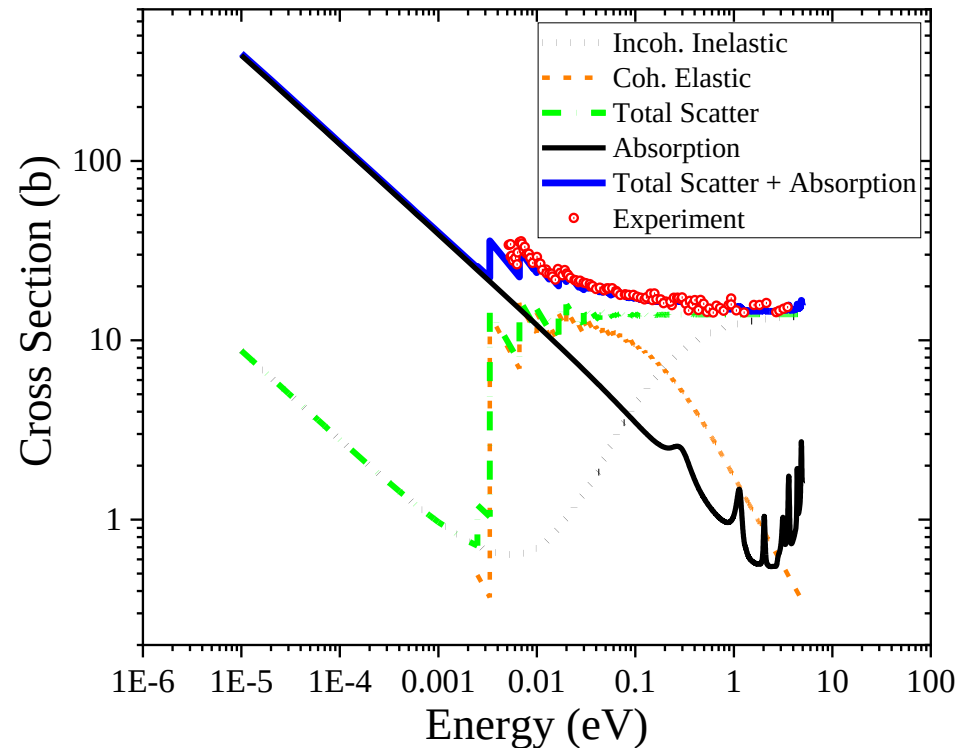
- Coherent elastic stored with uranium file
- Incoherent elastic stored with nitrogen
- Inelastic cross sections are a reflection of the unchanged DOS



Uranium Carbide

○ UC TSLs

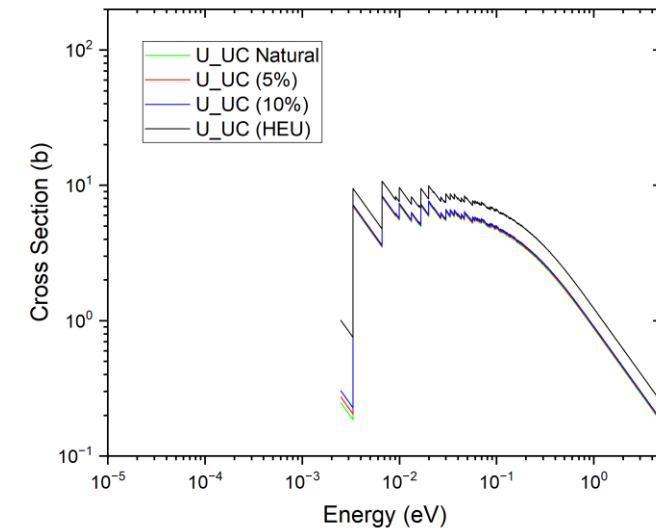
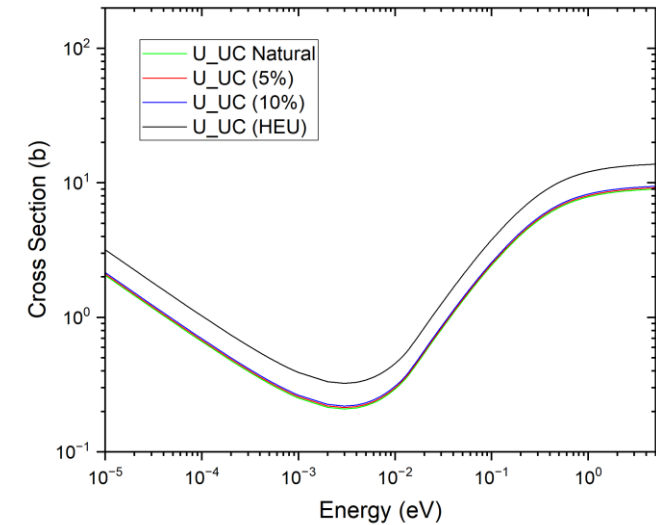
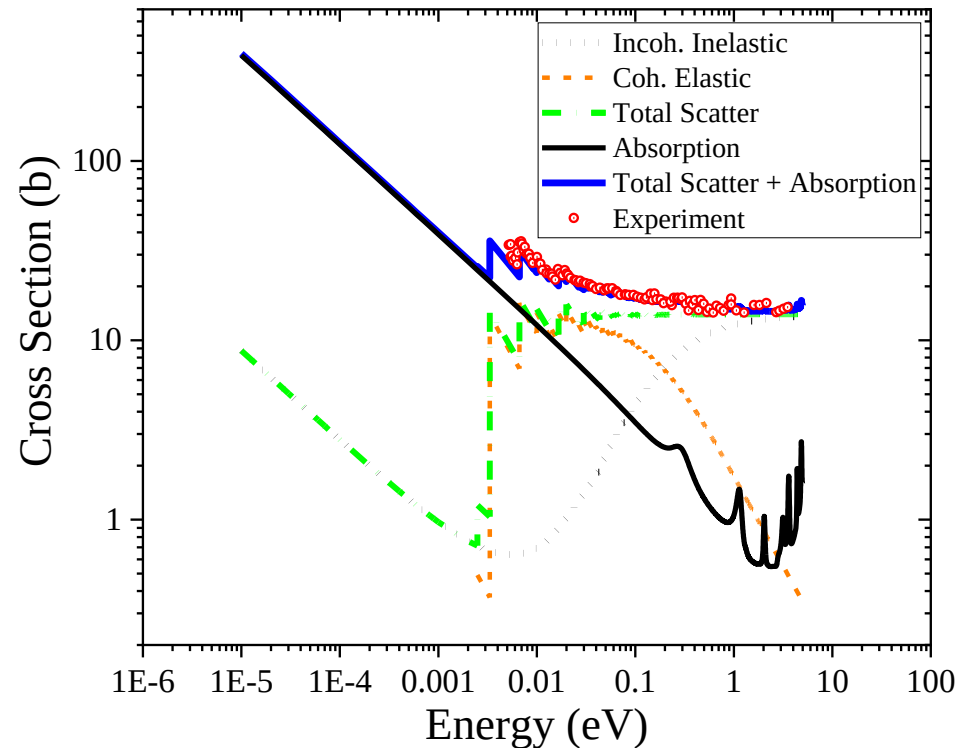
- Benchmarked against total cross section measurements
- Enrichment-dependent libraries available
 - Carbon and Uranium Libraries



Uranium Carbide

○ UC TSLs

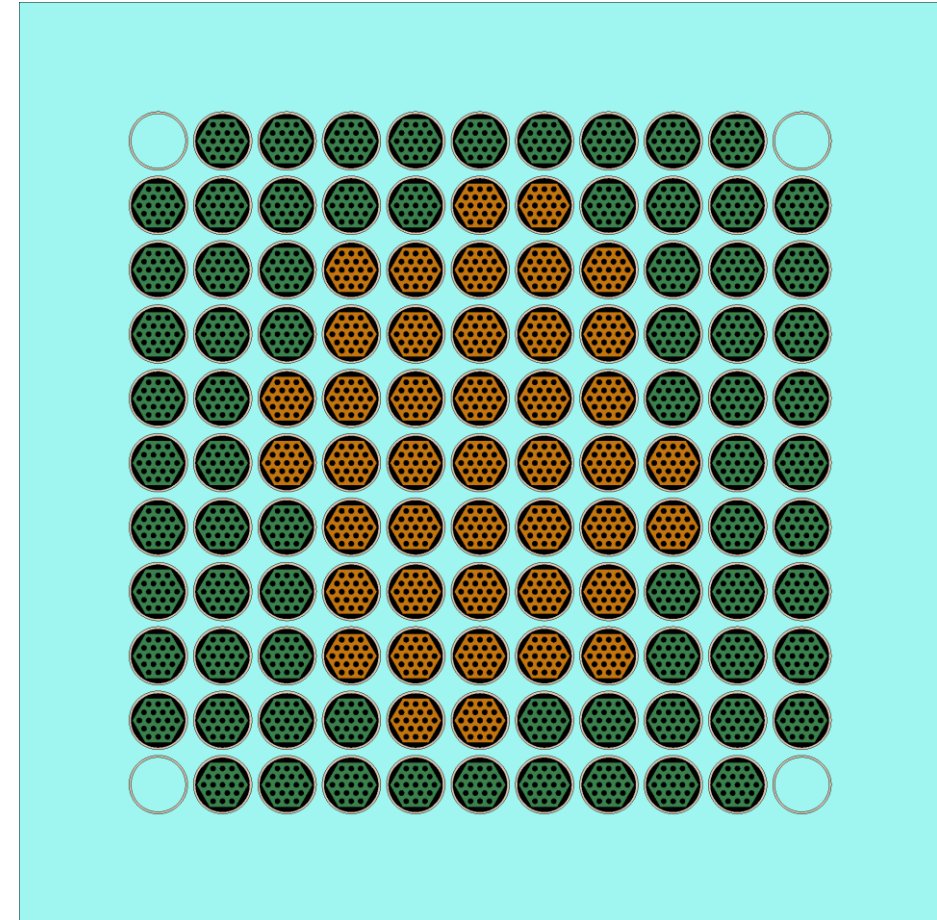
- Benchmarked against total cross section measurements
- Enrichment-dependent libraries available
 - Carbon and Uranium Libraries



Benchmark Applications

○ HEU-COMP-THERM-002

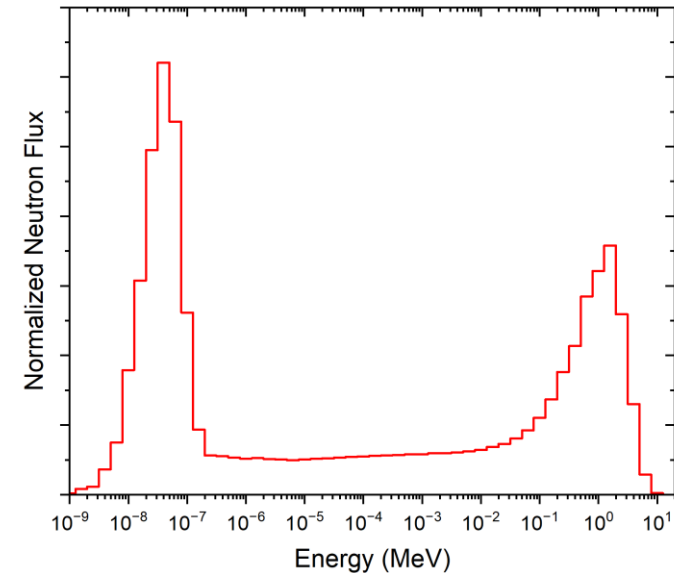
- Uranium carbide fuel matrix
(93.15% U-235 enriched)
- Water and polyethylene moderators
- Room temperature
- Thermal energy spectrum
- Other TSLs:
 - Light water
 - Polyethylene



Benchmark Applications

○ HEU-COMP-THERM-002

- Uranium carbide fuel matrix
(93.15% U-235 enriched)
- Water and polyethylene moderators
- Room temperature
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- Other TSLs:
 - Light water
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<u>Preliminary</u>	k_{eff}
Benchmark k_{eff}	1.0011 ± 0.0069
ENDF/B-VIII.0	1.01335 ± 0.0004
ENDF/B-VIII.0 + Natural UC TSL	1.01023 ± 0.0004
ENDF/B-VIII.0 + HEU UC TSL	1.00848 ± 0.0004
Impact from HEU TSL (pcm)	-487

Summary

New and updated uranium fuel evaluations have been submitted to the ENDF/B libraries. These include vital fuel materials U-metal, UC, UN, and UO₂.

- **Enrichment-Dependent Evaluations** – directly correlates to experimental data or benchmark for comparison
- **Improved Inputs** – corresponds with experimental structure and ENDF/B-VIII.0 accepted values for cross section and mass
- **QA Internal Checks Complete** – submitted to ENDF/B after passing through formal internal quality assurance checks

Thank You

