

Validation of ENDF/B-VIII.1- β 0-based Continuous Energy Data Tables

Noah Kleedtke*, Skip Kahler, Wim Haeck, Denise Neudecker,
Nathan Gibson

XCP-5: Materials and Physical Data
Los Alamos National Laboratory

2022 CSEWG Annual Meeting – Nuclear Data Validation

LA-UR-22-31596

*kleedtke@lanl.gov

Introduction

- Nuclear data library ENDF/B-VIII.1- $\beta 0$ files released early October 2022
- Changes from ENDF/B-VIII.0 →

- Designation of *a*, *b*, or *c* based on Pu-239 Evaluation:

(a) INDEN file ($\beta 0a.1$)

pu239p57

(b) LANL file from 05/25/2022 ($\beta 0b.1$)

lanl_05/25/2022_n_94-Pu-239.endf

(c) LLNL file ($\beta 0c.1$)

yi01za094239.endf/2022-ex4+0.17pc

- This analysis also includes
an updated LANL evaluation from
10/17/2022 ($\beta 0b.2$)

lanl_10/17/2022_n_94-Pu-239.endf

- 28,29,30Si taken from INDEN: ORNL/LTR-2018/1044 (M. Pigni et al., 2018)
- 50,53,54Cr taken from INDEN: v2.3.2, NDS 173, 1-41 (2021)
- 52Cr taken from INDEN: v2.3.4 (cr52v234c), NDS 173, 1-41 (2021) + RRR changes
- 54Fe taken from INDEN: fe54e80p
- 56Fe taken from INDEN: fe56e80X29r50
- 57Fe taken from INDEN: fe57e80o
- 63Cu taken from INDEN: cu63ane6k09aRR
- 65Cu taken from INDEN: cu65ane5k05
- 233U taken from INDEN: u233ornl_2b_nu12
- 235U taken from INDEN: u235ib46o28t6DNcnu5ef0ST25d
- 238U taken from INDEN (with Q-value fix): u238e80u8V02b + GitLab commit ff23a85b

- Continuous-energy tables were processed with NJOY2016 into ACE
(a continuous energy) formatted files
- Validation was performed using ICSBEP benchmarks [1] with the MCNP®
code, version 6.2 [2]

Background

- **Neutron Reactor Kinetics Parameters:**

- (1) Prompt neutron decay constant (α) – measures time rate of change of the prompt neutron population

$$\alpha = \frac{k_p - 1}{l}$$

k_p : prompt neutron multiplication factor, l : mean neutron lifetime

- (2) Effective Delayed Neutron Fraction (β_{eff}) – measures effectiveness of the delayed neutron population causing fission

$$\beta_{\text{eff}} = 1 - \frac{k_p}{k}$$

- **Validation Metrics:**

- Bias = Calculated Value (C) – Experimental Value (E)
- Mean Absolute Bias (“Mean Abs. Bias” in figures) = $\frac{|C-E|}{n}$

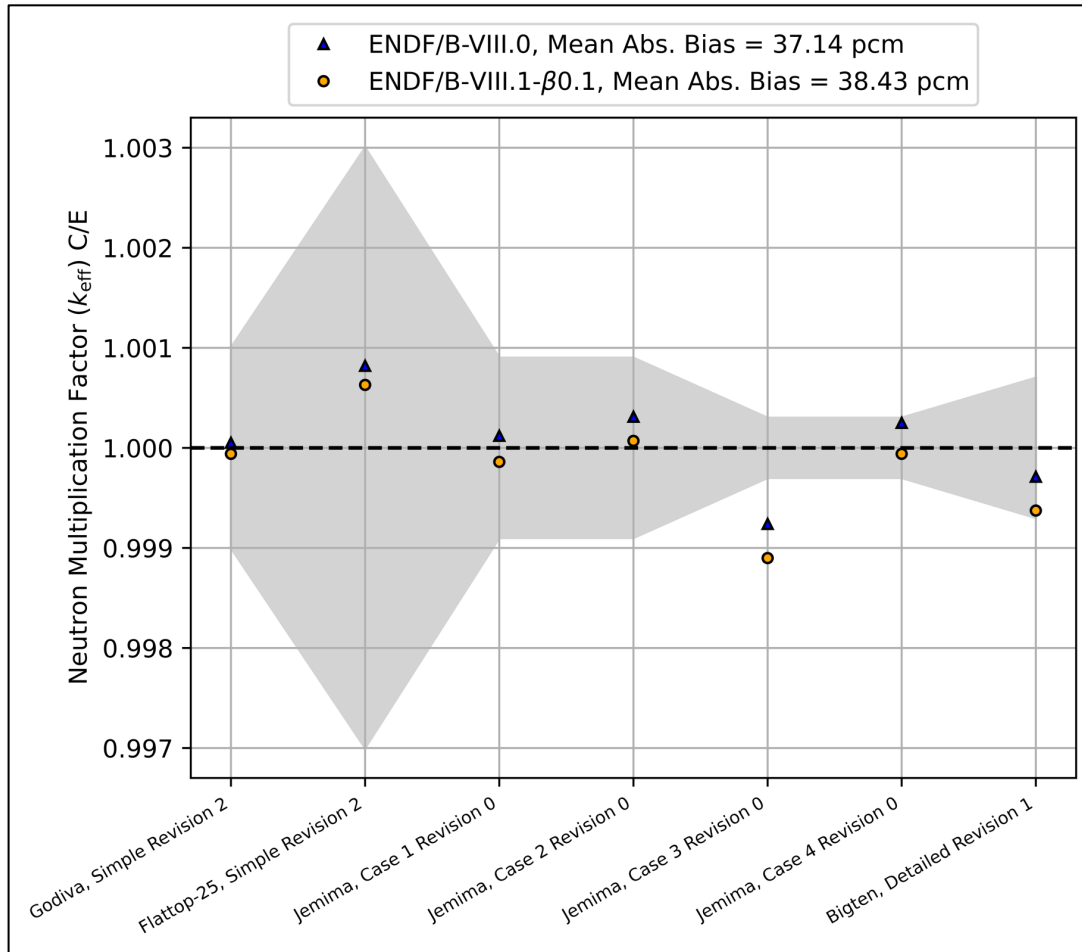
n : number of benchmark experiments

- **Relative Sensitivity Coefficient Calculation:**

σ : some nuclear data

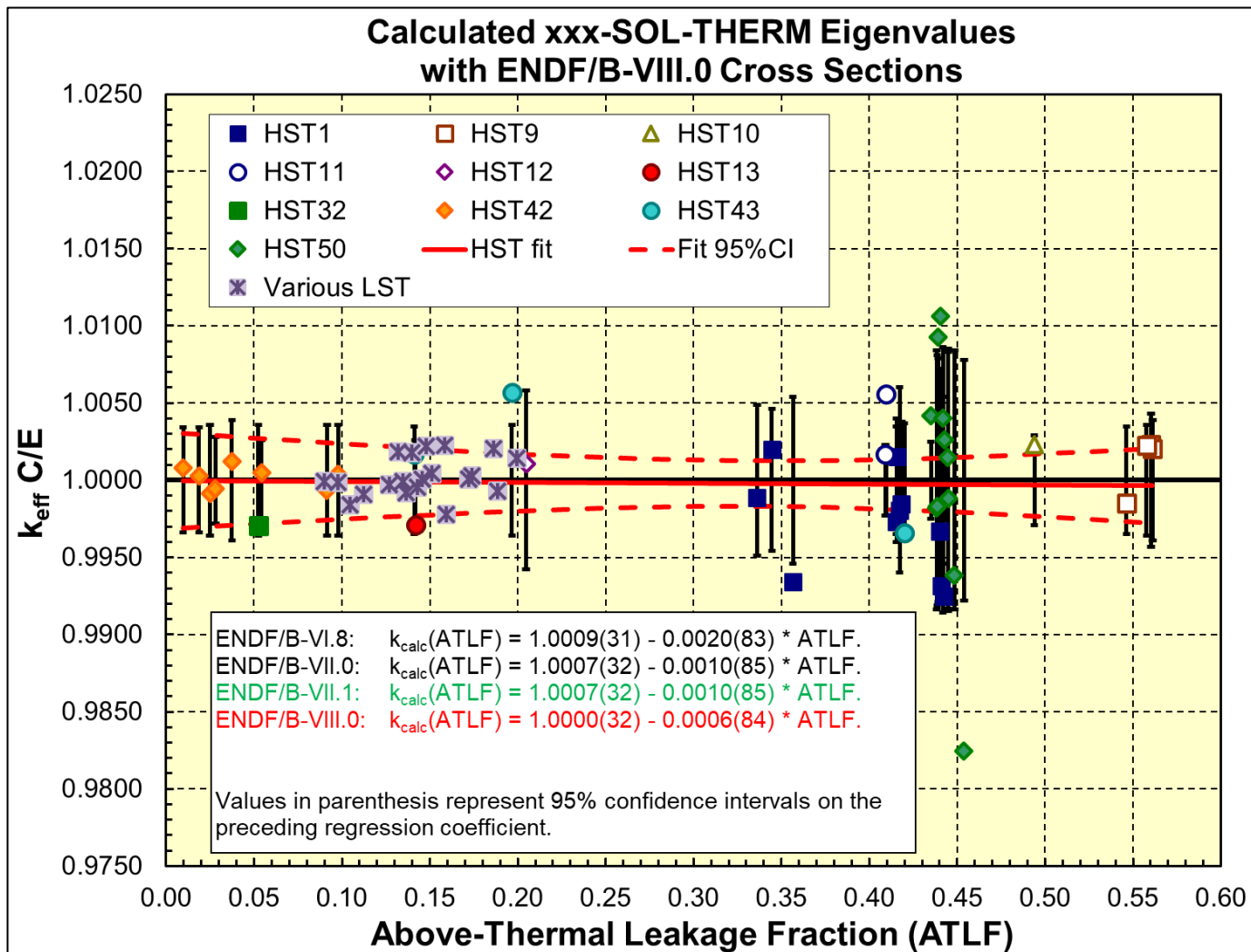
$$S_{k_{\text{eff}},\sigma} = \frac{\sigma}{k_{\text{eff}}} \frac{\partial k_{\text{eff}}}{\partial \sigma}$$

Results – ENDF/B-VIII.0 $S(\alpha, \beta)$, ENDF/B-VIII- $\beta 0$ nuclides, Non-Pu Legacy Benchmark Suite



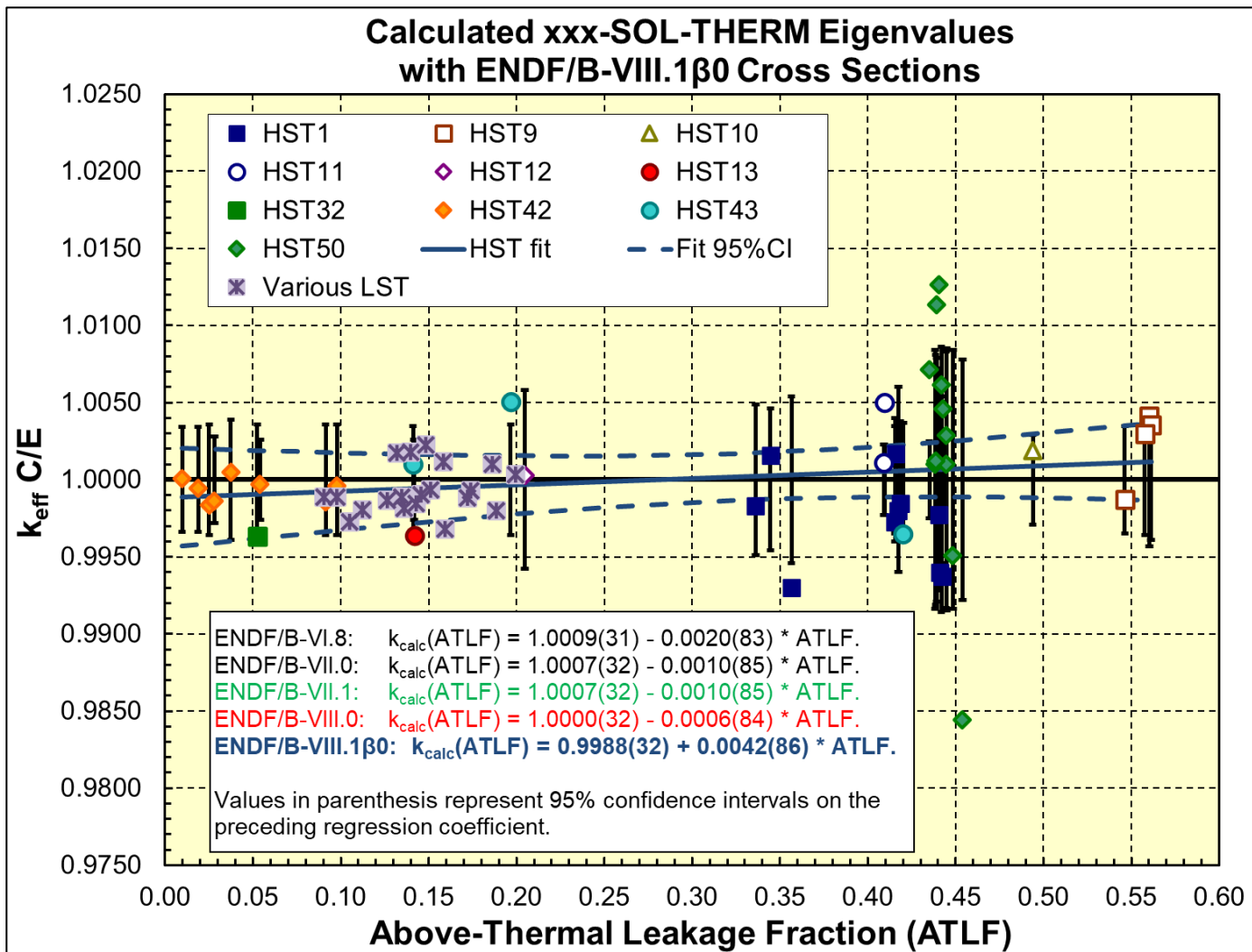
Benchmark	Bias (pcm)	
	ENDF/B-VIII.0	ENDF/B-VIII.1- $\beta 0.1$
Godiva	<u>+5</u>	-6
Flattop-25	+82	<u>+63</u>
Jemima Case 1	<u>+12</u>	-14
Jemima Case 2	+31	<u>+7</u>
Jemima Case 3	<u>-76</u>	-110
Jemima Case 4	+25	<u>-6</u>
Bigten	<u>-29</u>	-63

Results – ENDF/B-VIII.0 $S(\alpha, \beta)$, ENDF/B-VIII- $\beta 0$ nuclides



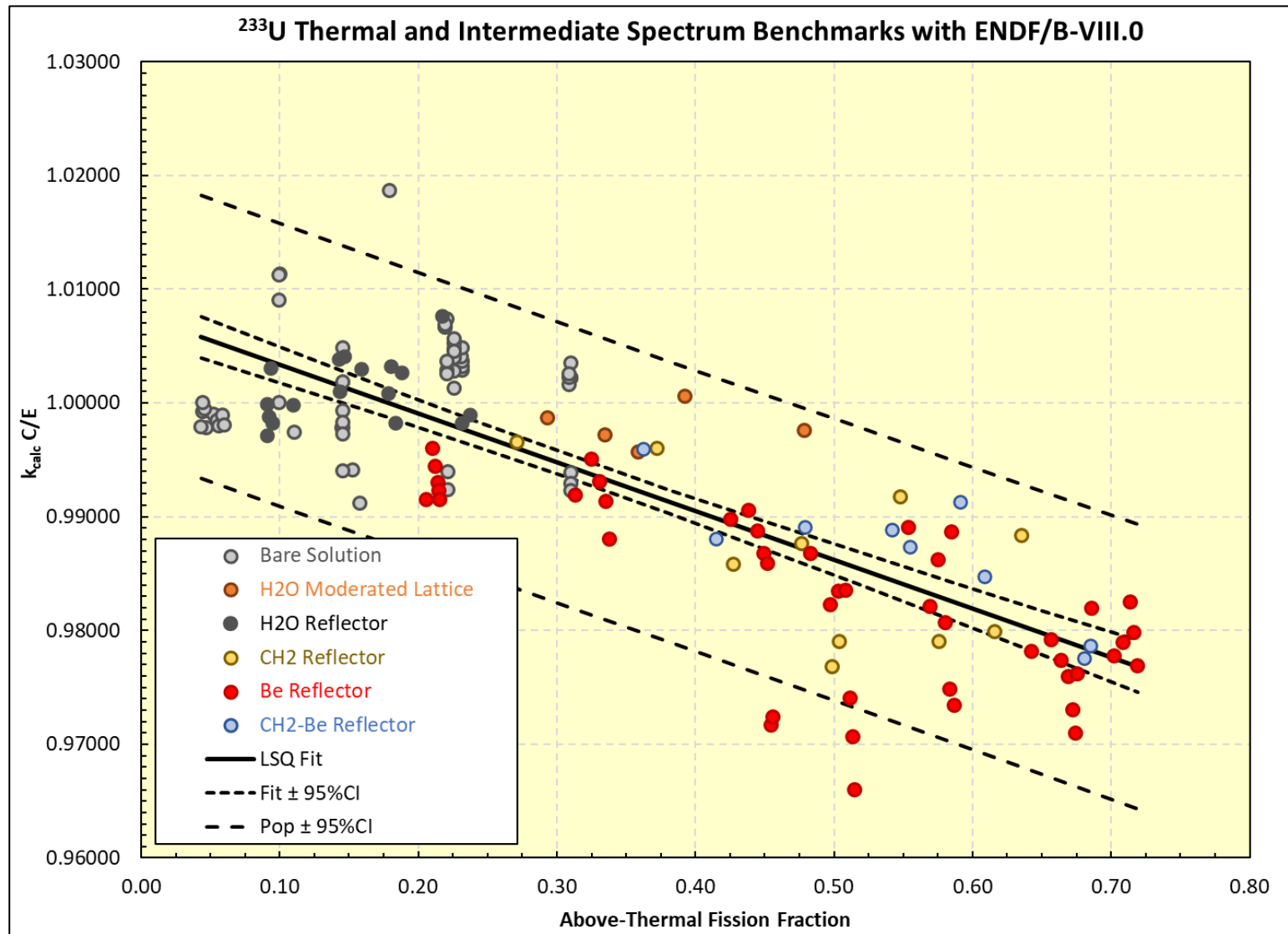
HST-01: Uranyl Nitrate
 HST-09: Uranium Oxyfluoride
 HST-10: Uranium Oxyfluoride
 HST-11: Uranium Oxyfluoride
 HST-12: Uranium Oxyfluoride
 HST-13: Uranium Nitrate
 HST-32: Uranyl Nitrate
 HST-42: Uranium Nitrate
 HST-43: Uranium Oxyfluoride
 HST-50: Uranyl Fluoride

Results – ENDF/B-VIII.0 $S(\alpha, \beta)$, ENDF/B-VIII- $\beta 0$ nuclides

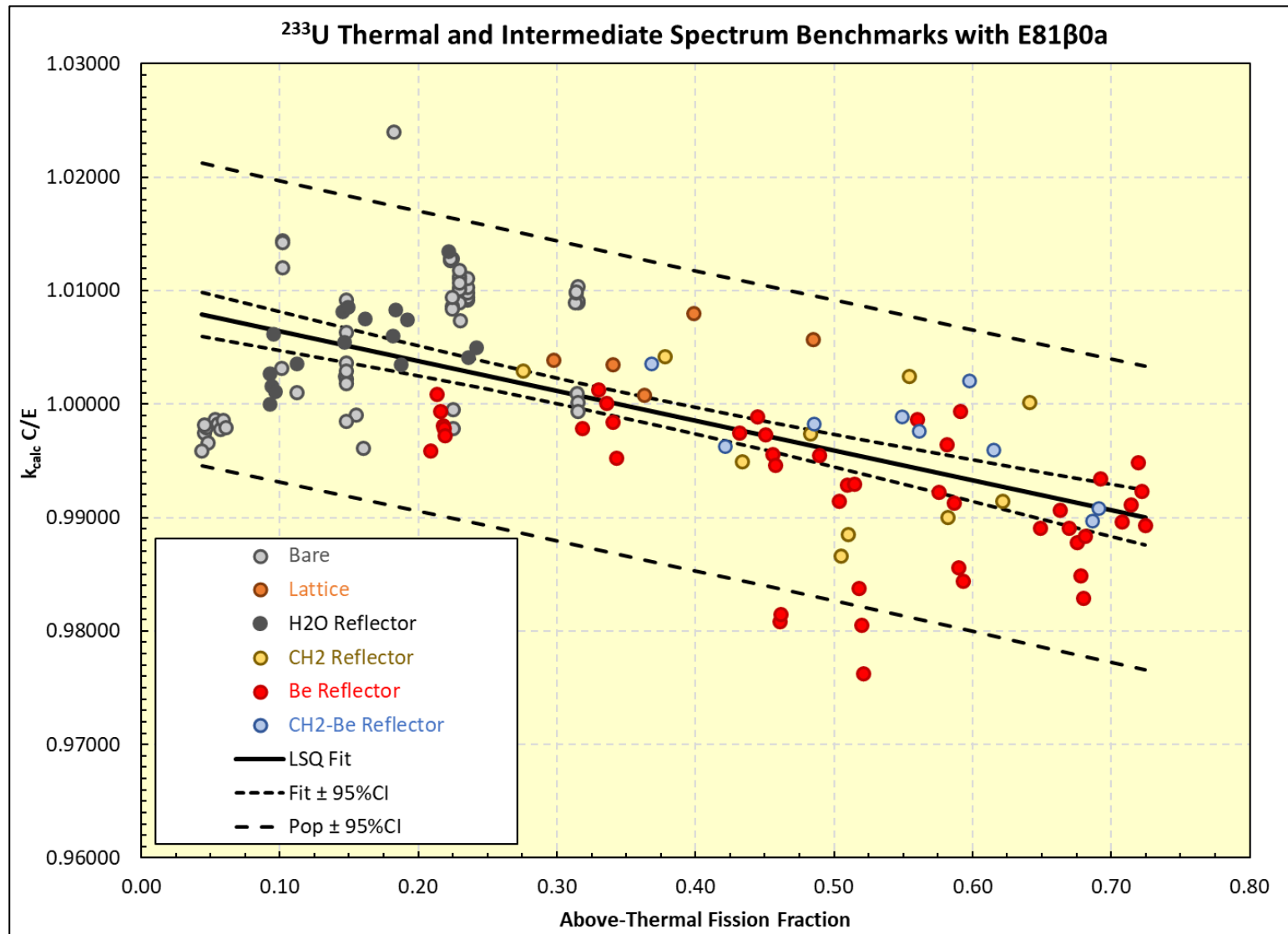


HST-01: Uranyl Nitrate
 HST-09: Uranium Oxyfluoride
 HST-10: Uranium Oxyfluoride
 HST-11: Uranium Oxyfluoride
 HST-12: Uranium Oxyfluoride
 HST-13: Uranium Nitrate
 HST-32: Uranyl Nitrate
 HST-42: Uranium Nitrate
 HST-43: Uranium Oxyfluoride
 HST-50: Uranyl Fluoride

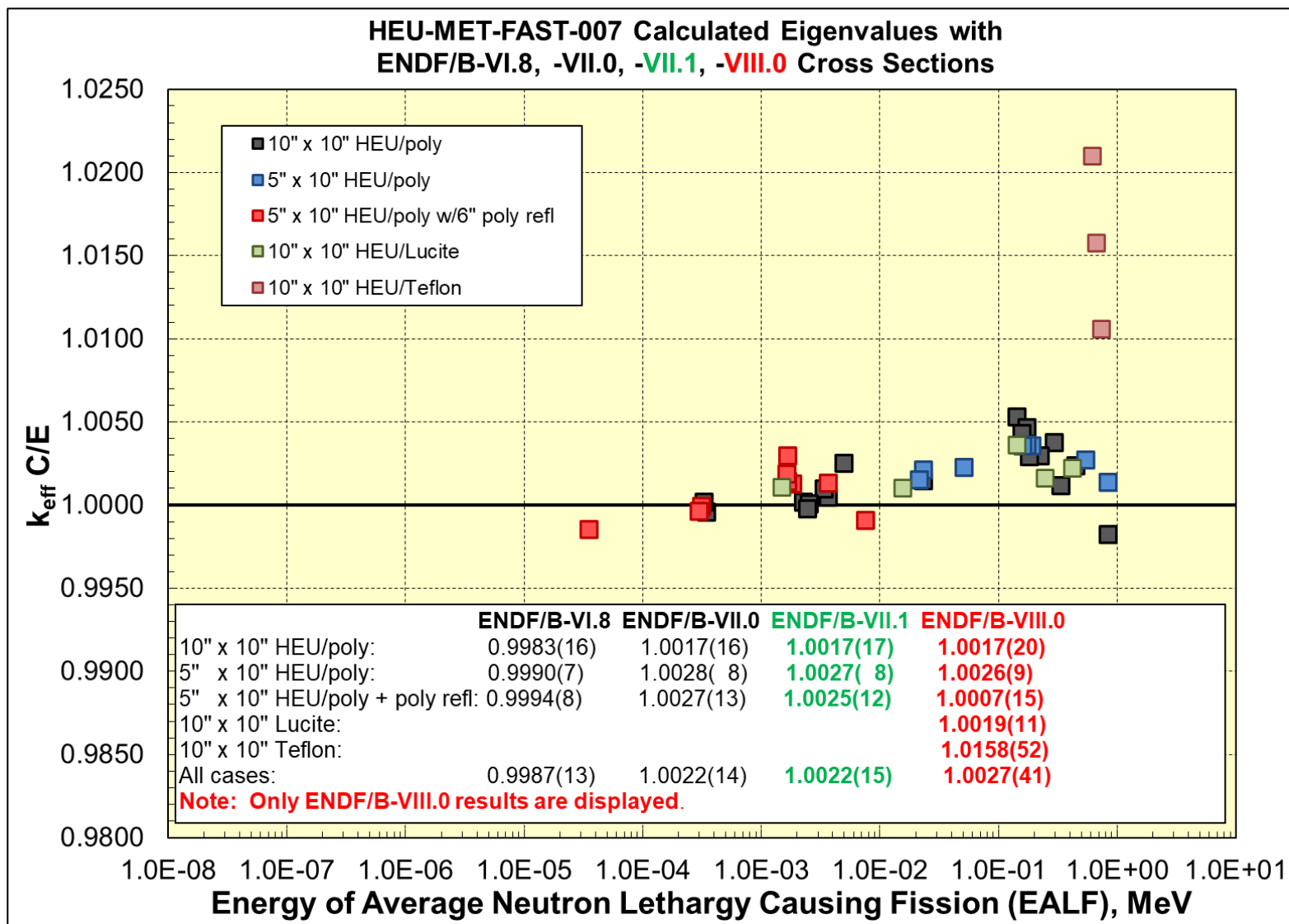
Results – ENDF/B-VIII.0 $S(\alpha, \beta)$, ENDF/B-VIII- $\beta 0$ nuclides



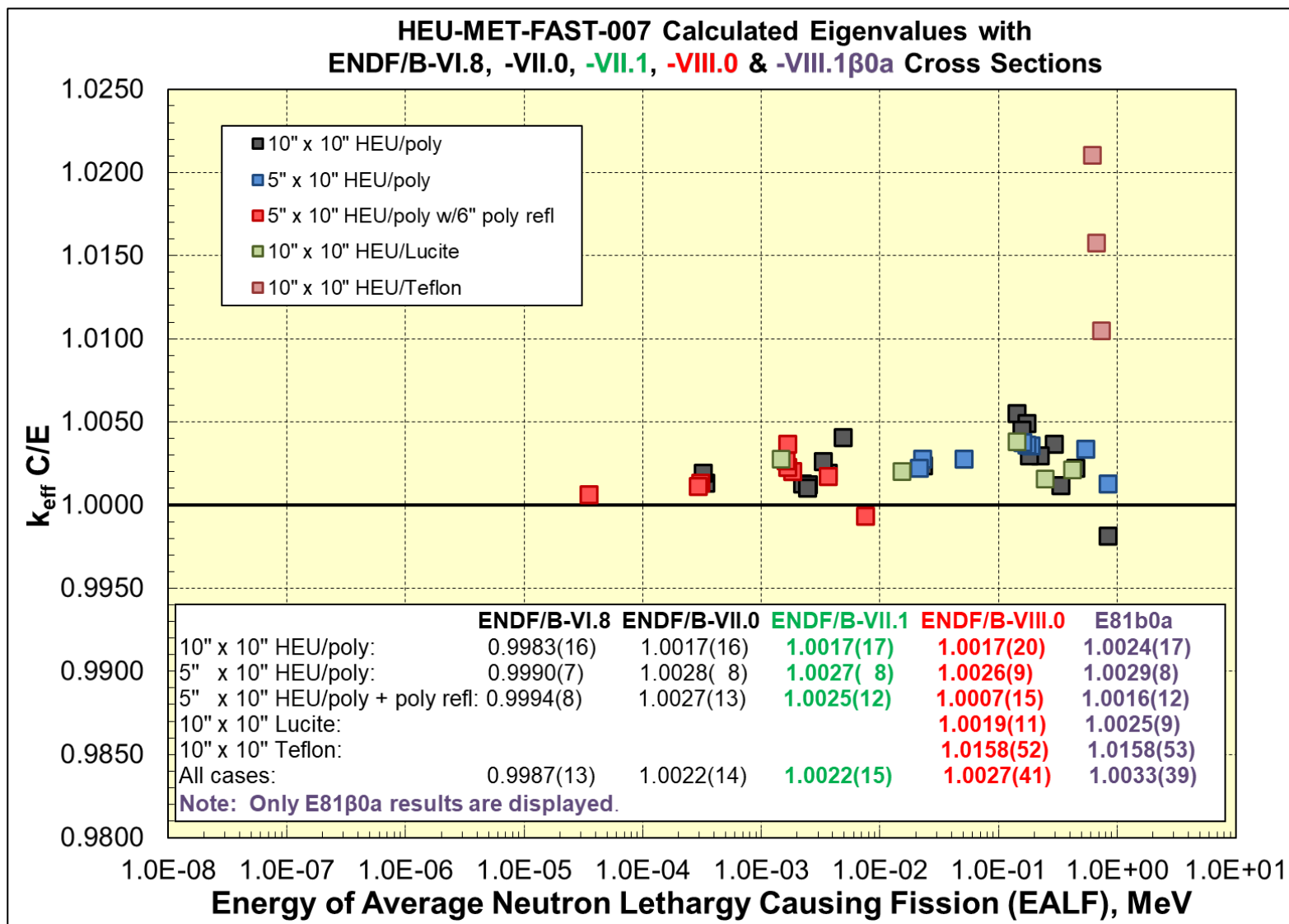
Results – ENDF/B-VIII.0 $S(\alpha, \beta)$, ENDF/B-VIII- $\beta 0$ nuclides



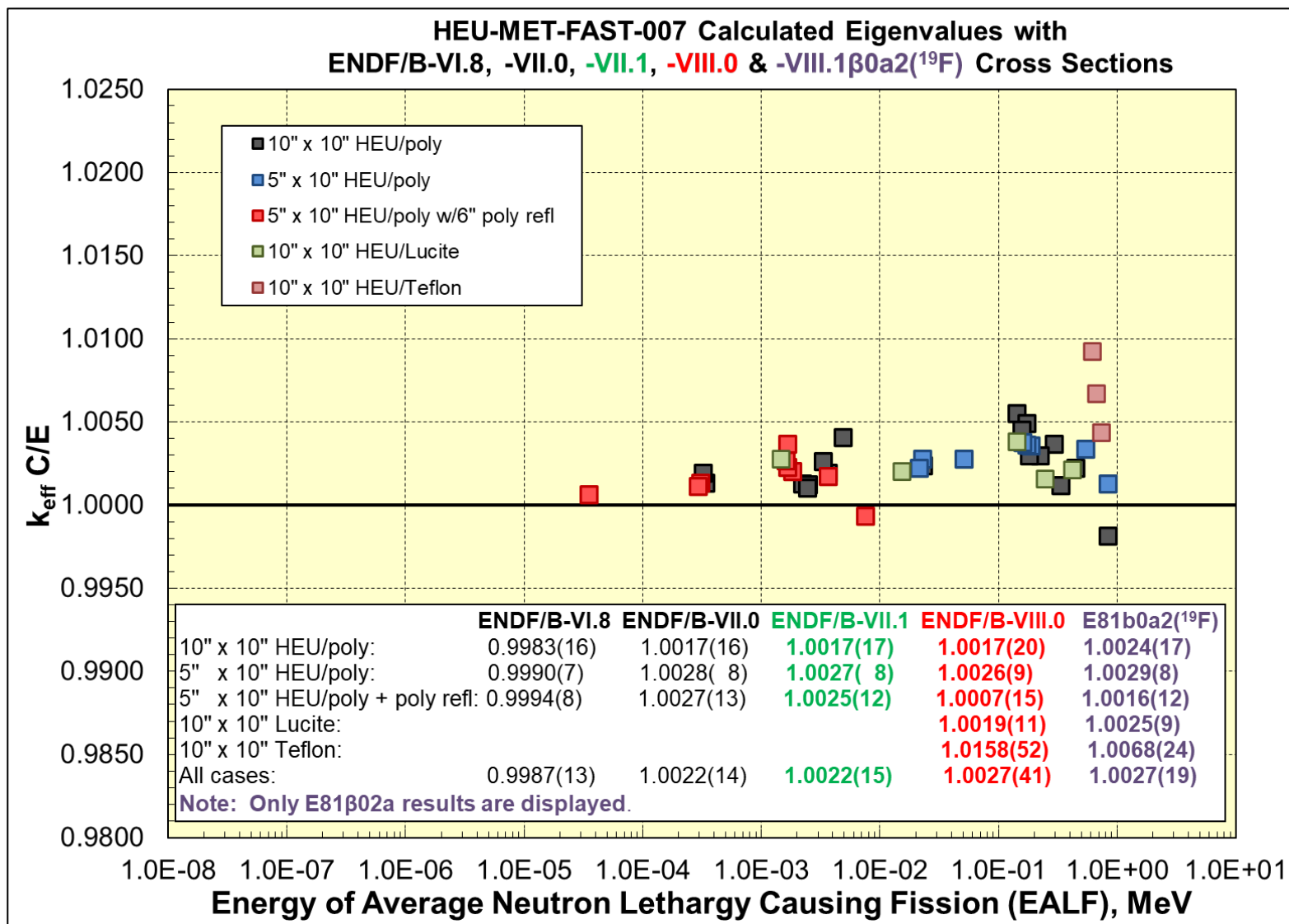
Results – ENDF/B-VIII.0 $S(\alpha, \beta)$, ENDF/B-VIII- $\beta 0$ nuclides



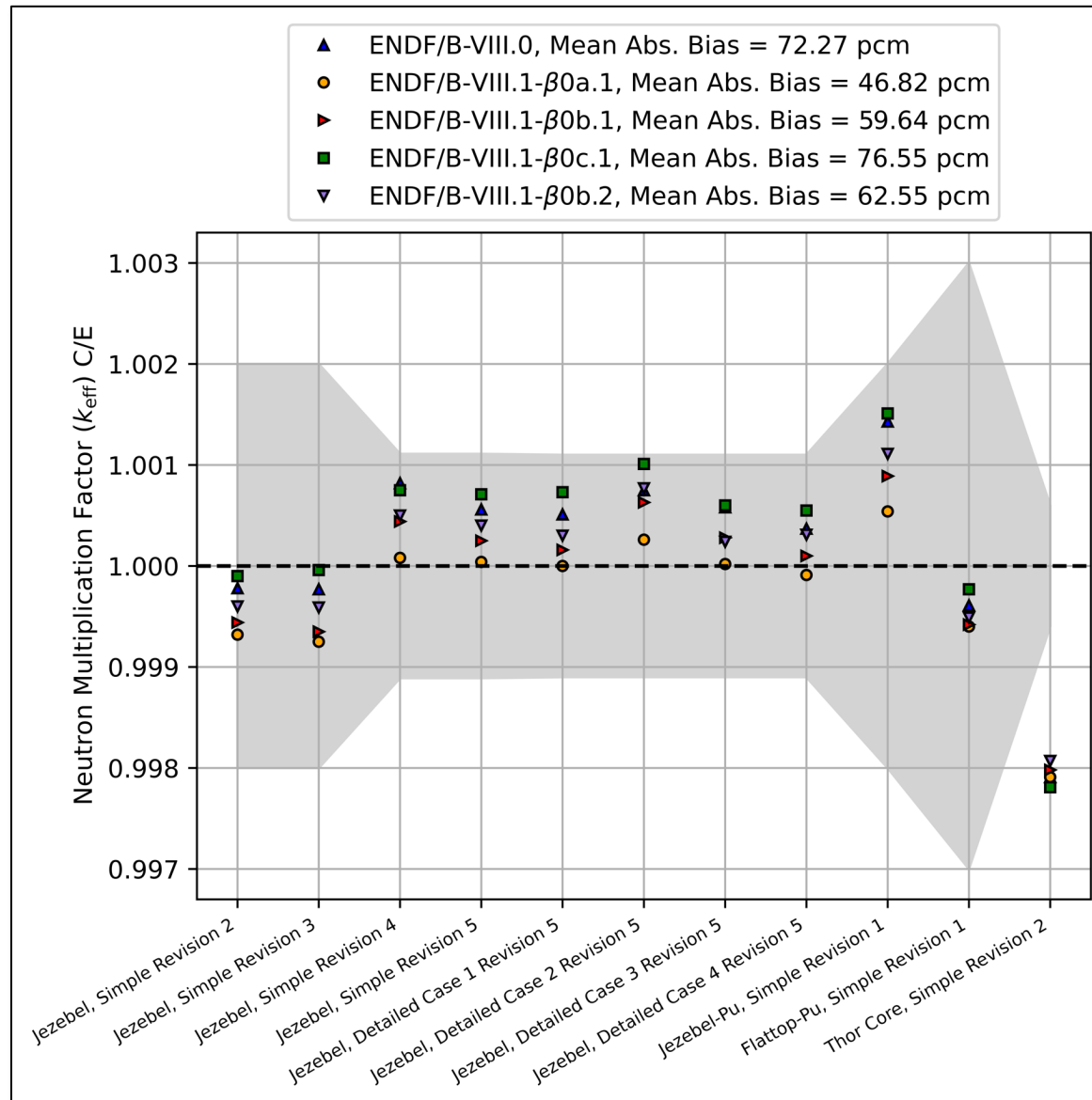
Results – ENDF/B-VIII.0 $S(\alpha, \beta)$, ENDF/B-VIII- $\beta 0$ nuclides



Results – ENDF/B-VIII.0 $S(\alpha, \beta)$, ENDF/B-VIII- $\beta 0$ nuclides



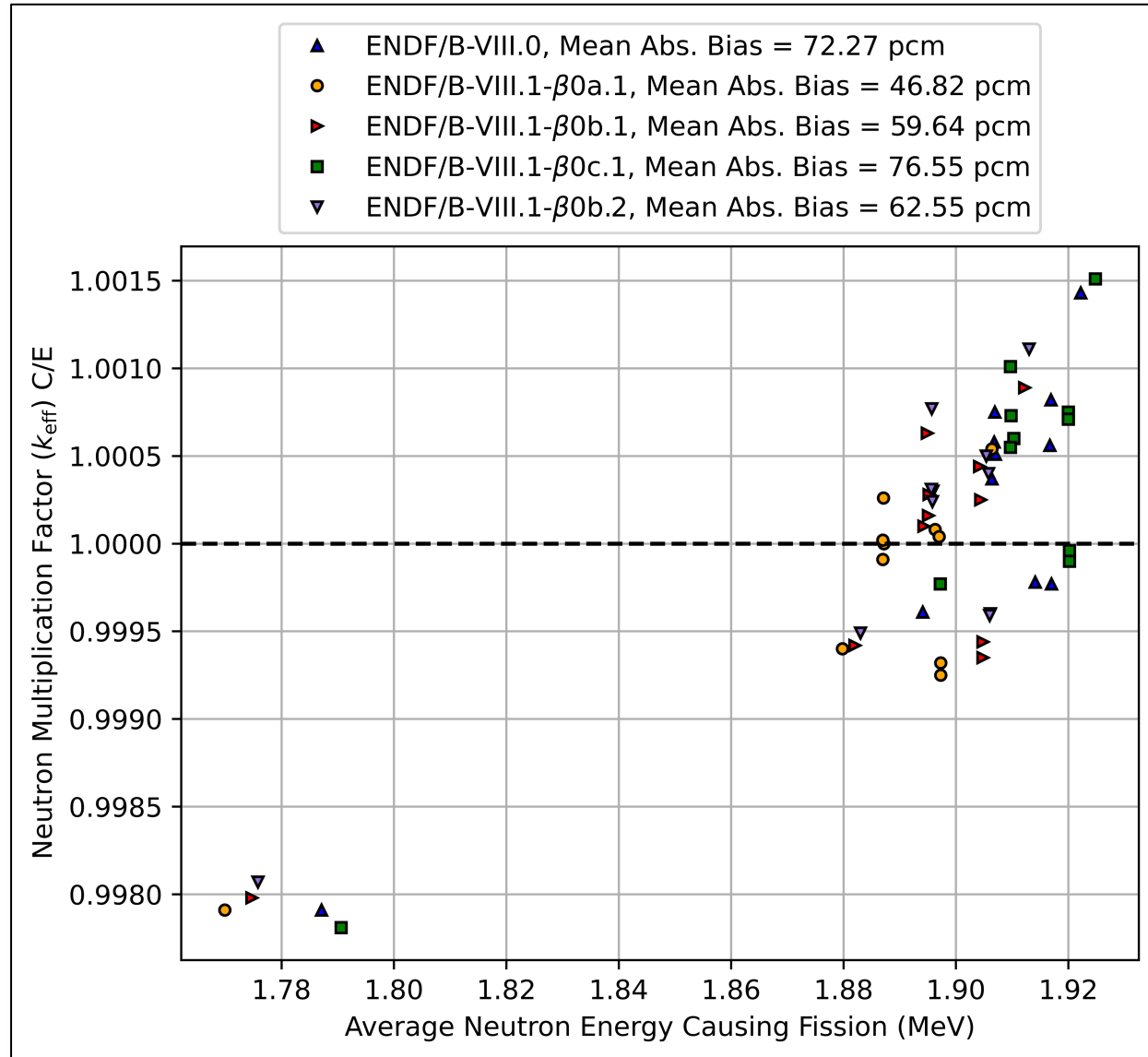
Results – ENDF/B-VIII.0 $S(\alpha, \beta)$, ENDF/B-VIII- $\beta 0$ nuclides, Pu Legacy Benchmark Suite



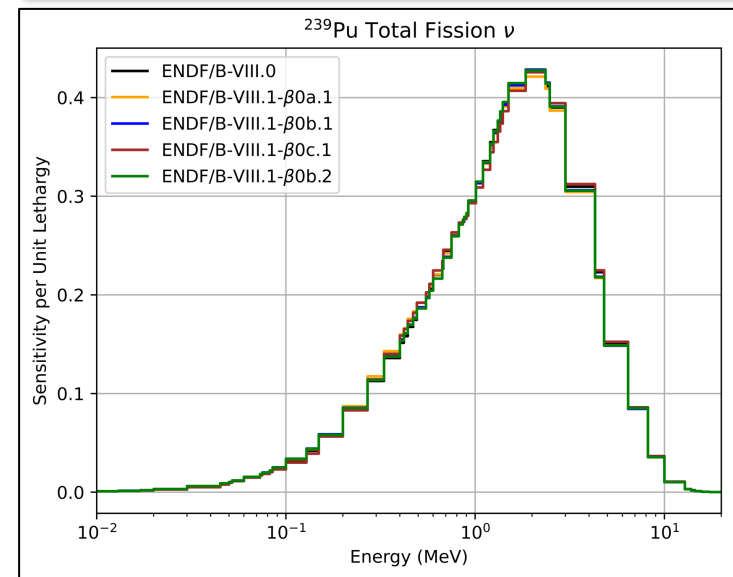
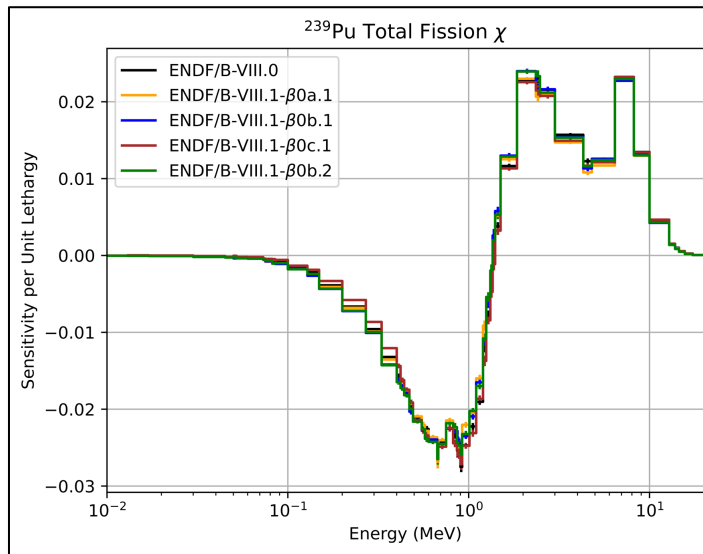
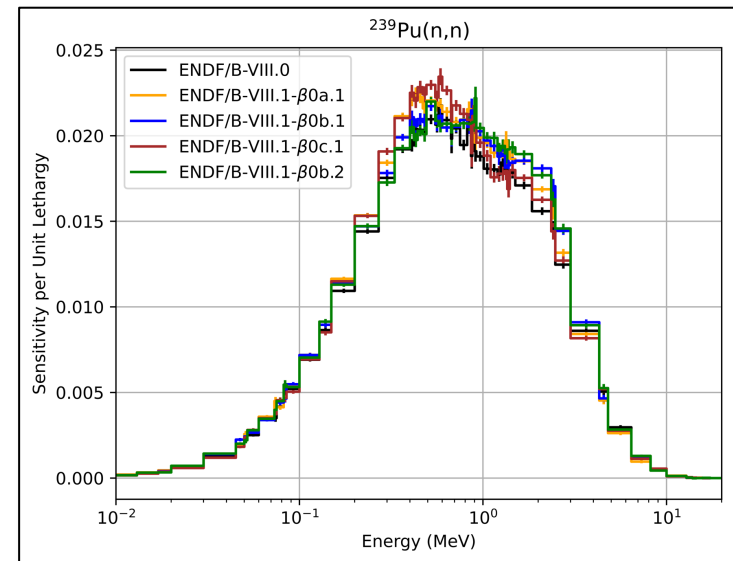
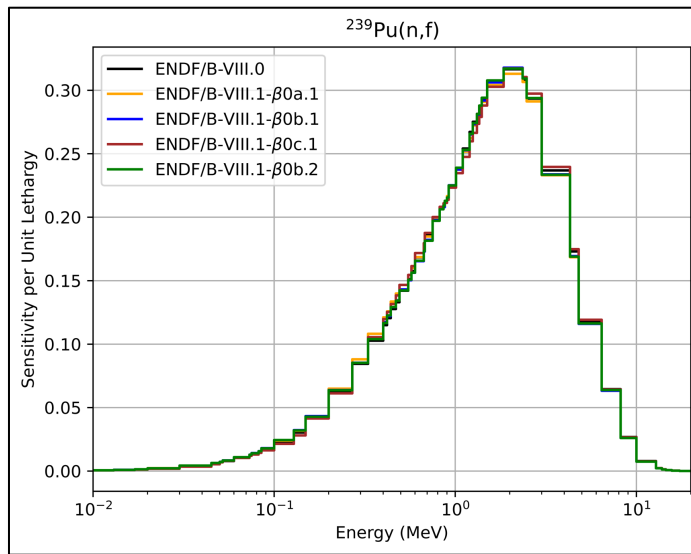
Results – ENDF/B-VIII.0 $S(\alpha, \beta)$, ENDF/B-VIII- $\beta 0$ nuclides, Pu Legacy Benchmark Suite

Benchmark	Bias (pcm)				
	ENDF/B-VIII.0	ENDF/B-VIII.1- $\beta 0a.1$	ENDF/B-VIII.1- $\beta 0b.1$	ENDF/B-VIII.1- $\beta 0c.1$	ENDF/B-VIII.1- $\beta 0b.2$
Jezebel Simple rev2	-22	-68	-56	<u>-10</u>	-40
Jezebel Simple rev3	-23	-75	-65	<u>-4</u>	-41
Jezebel Simple rev4	+82	<u>+8</u>	+44	+75	+50
Jezebel Simple rev5	+56	<u>+4</u>	+25	+71	+40
Jezebel Detailed rev5 Case 1	+51	<u>0</u>	+16	+73	+30
Jezebel Detailed rev5 Case 2	+75	<u>+26</u>	+63	+101	+77
Jezebel Detailed rev5 Case 3	+58	<u>+2</u>	+28	+60	+24
Jezebel Detailed rev5 Case 4	+37	<u>-9</u>	+10	+55	+31
Jezebel-Pu ("Dirty Jezebel")	+143	<u>+54</u>	+89	+151	+111
Flattop-Pu	-39	-60	-58	<u>-23</u>	-51
Thor Core	-209	-209	-202	-219	<u>-193</u>

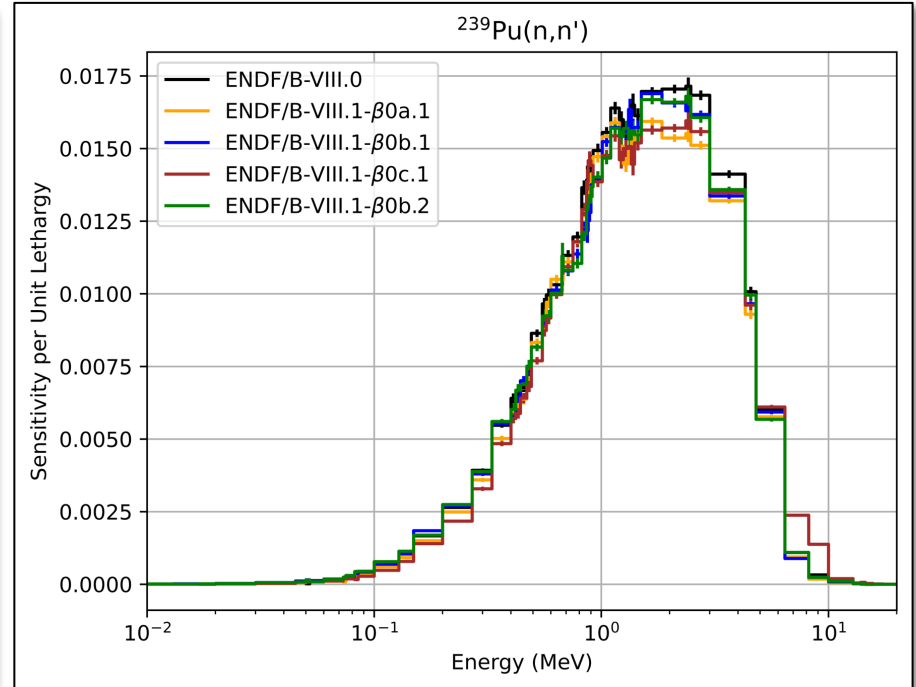
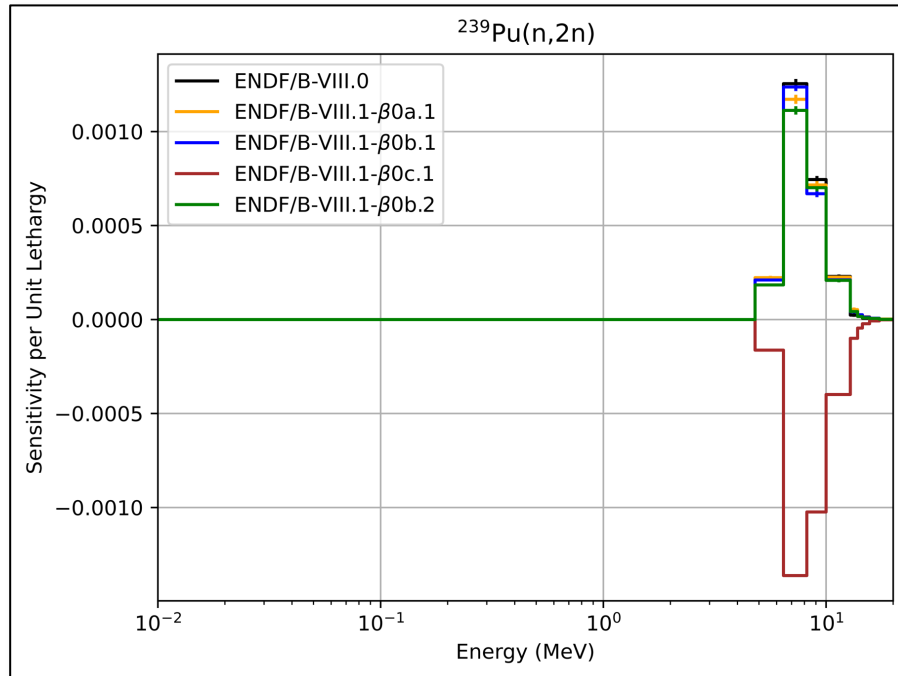
Results – ENDF/B-VIII.0 $S(\alpha, \beta)$, ENDF/B-VIII- $\beta 0$ nuclides, Pu Legacy Benchmark Suite



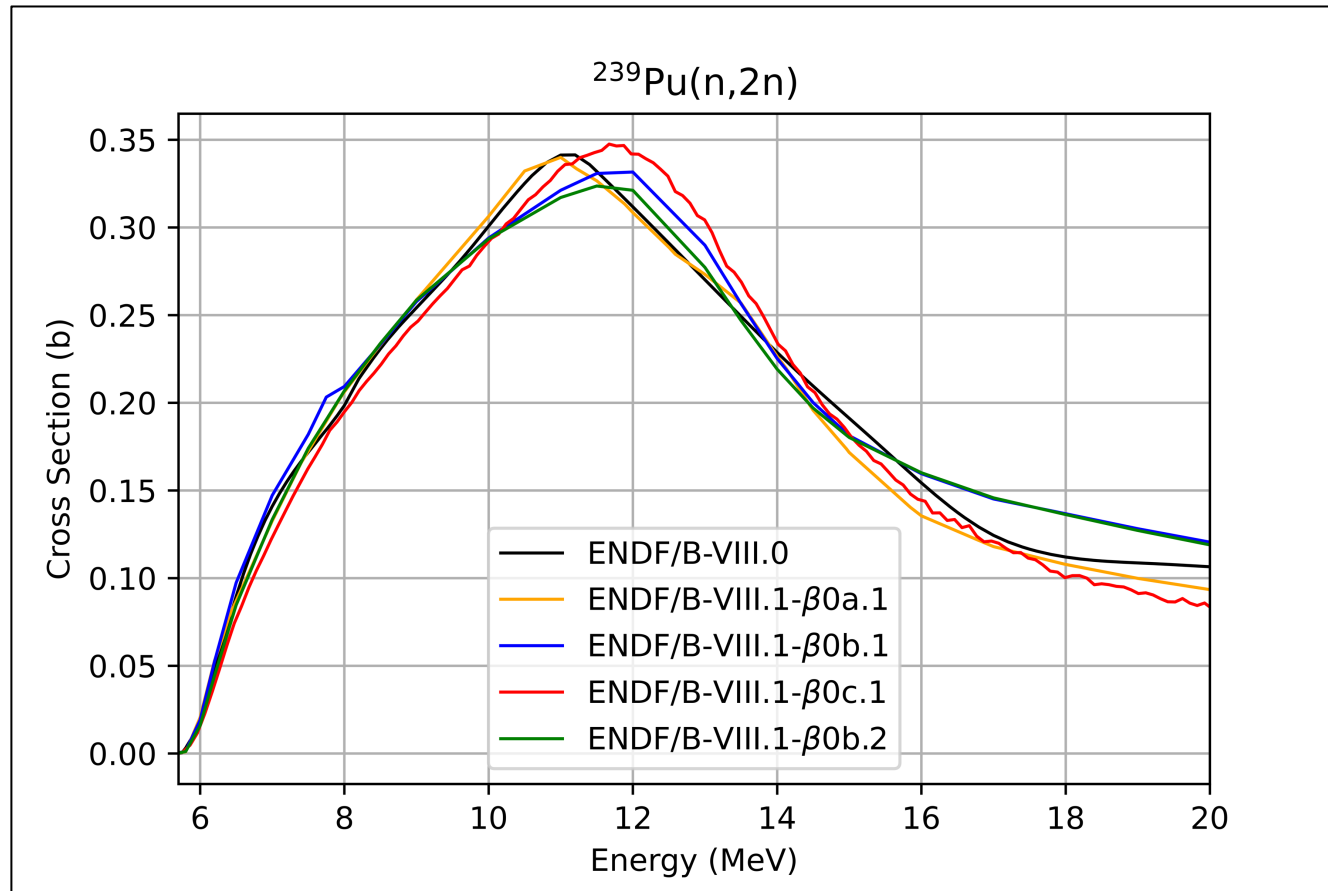
Results – ENDF/B-VIII.0 $S(\alpha, \beta)$, ENDF/B-VIII- $\beta 0$ nuclides, Jezebel Simple rev5 Benchmark ^{239}Pu Sensitivity



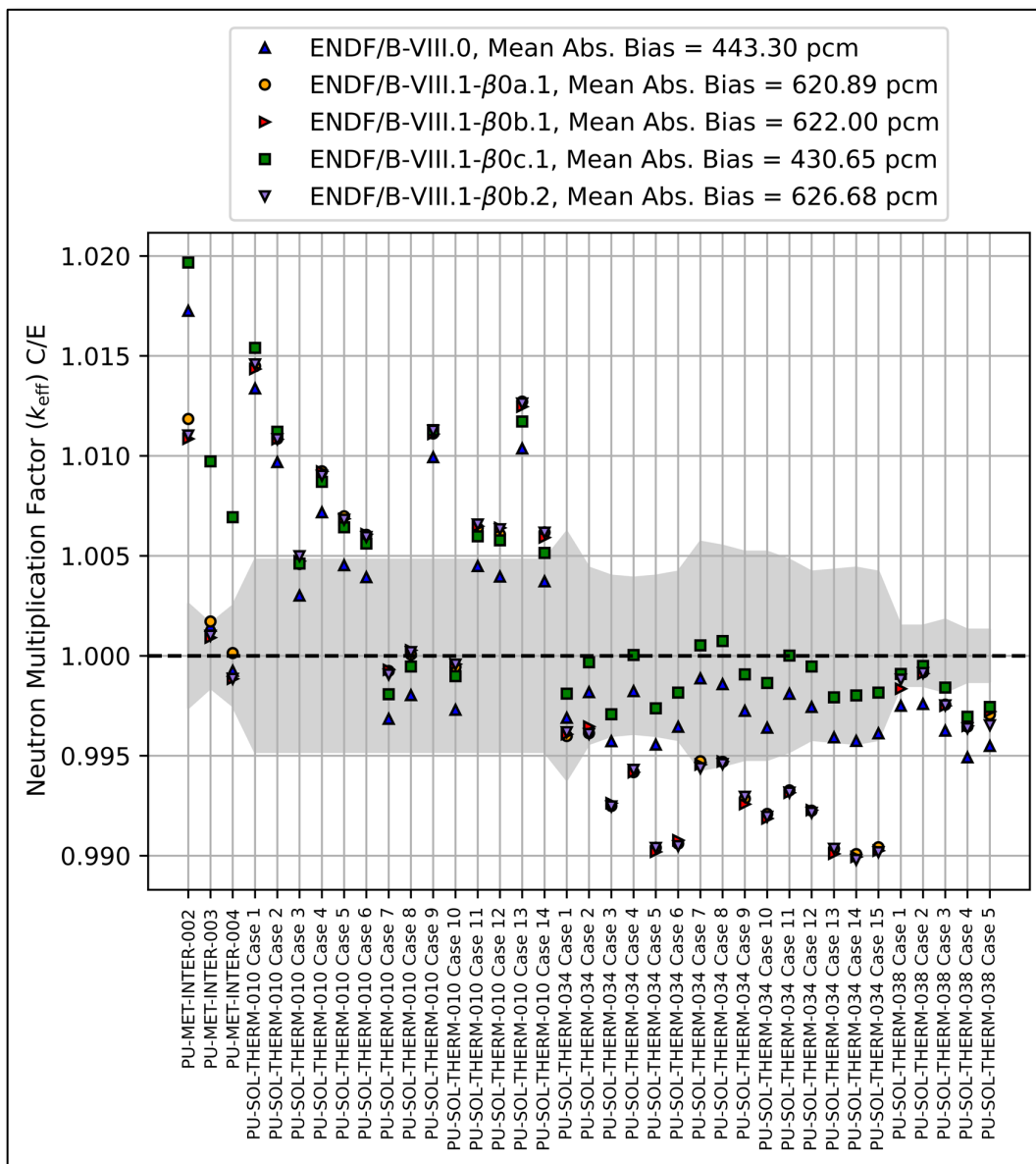
Results – ENDF/B-VIII.0 $S(\alpha, \beta)$, ENDF/B-VIII- $\beta 0$ nuclides, Jezebel Simple rev5 Benchmark ^{239}Pu Sensitivity



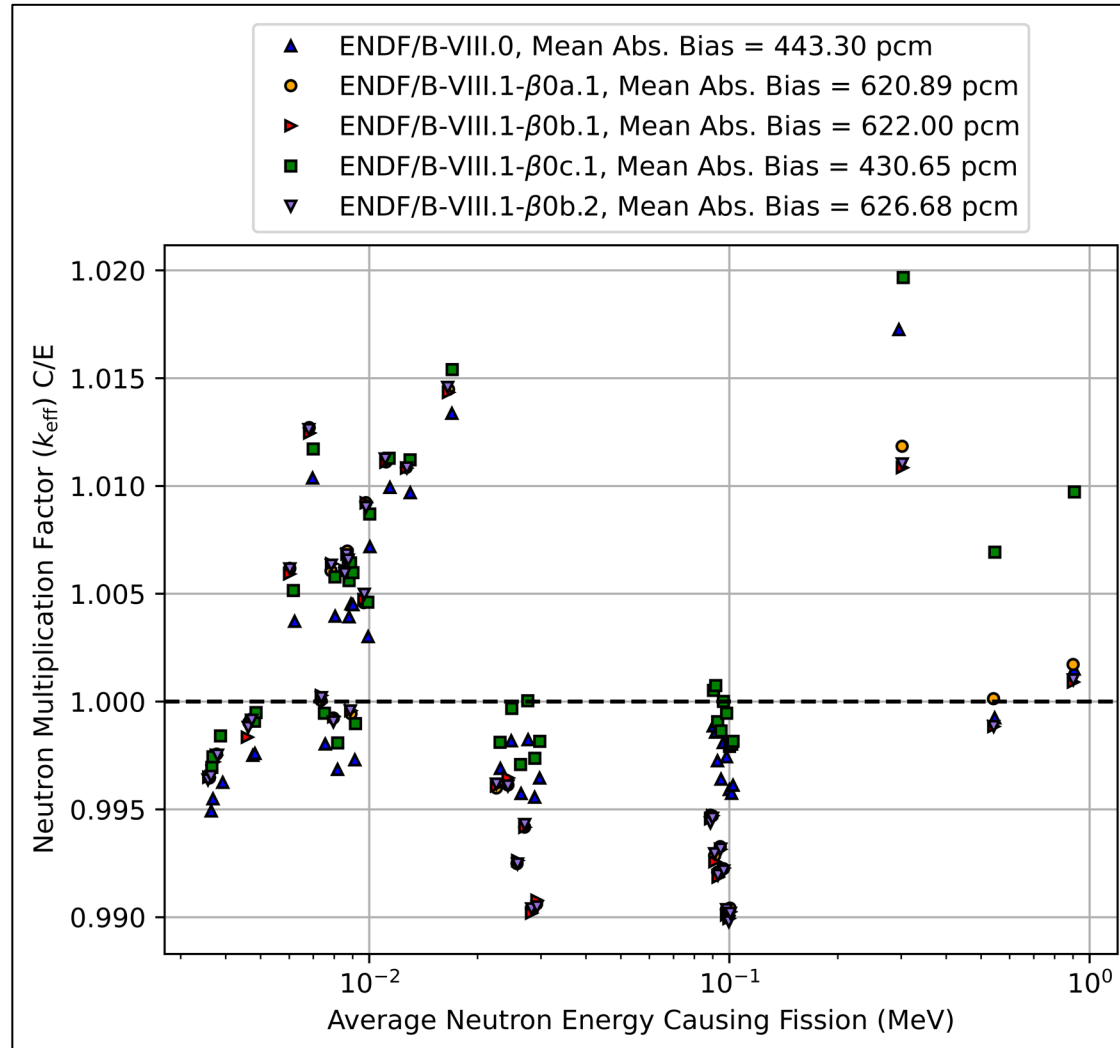
Results – ENDF/B-VIII.0 $S(\alpha, \beta)$, ENDF/B-VIII- $\beta 0$ nuclides



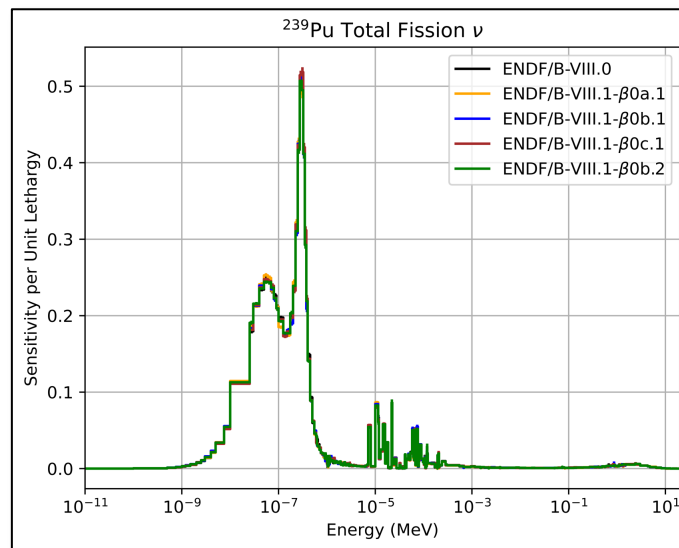
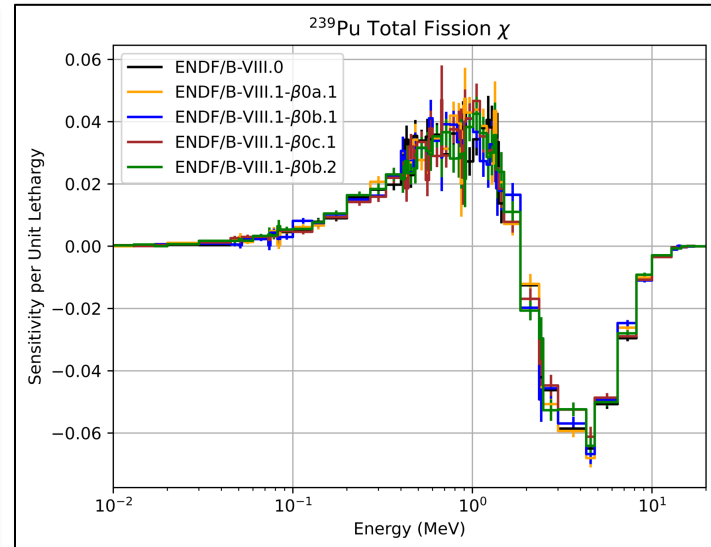
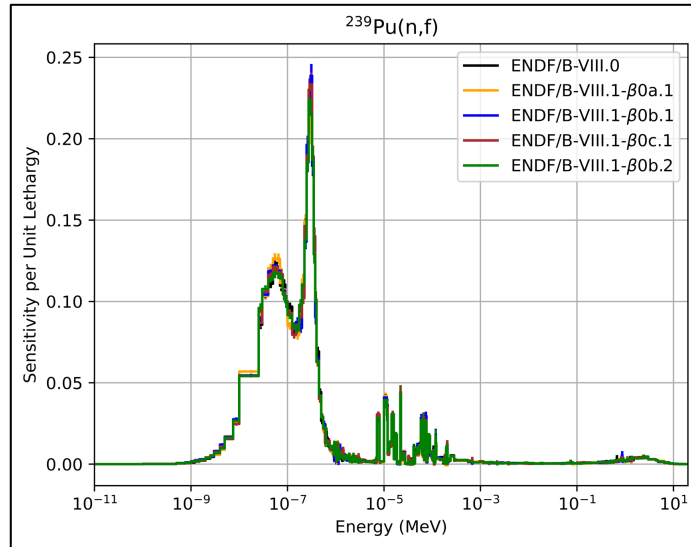
Results – ENDF/B-VIII.0 $S(\alpha, \beta)$, ENDF/B-VIII- $\beta 0$ nuclides, PU-MET-INTER/PU-SOL-THERM Benchmark Suite



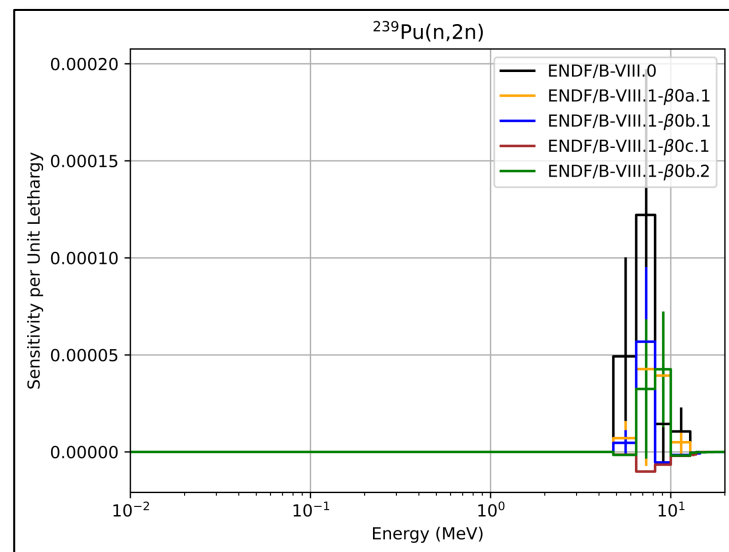
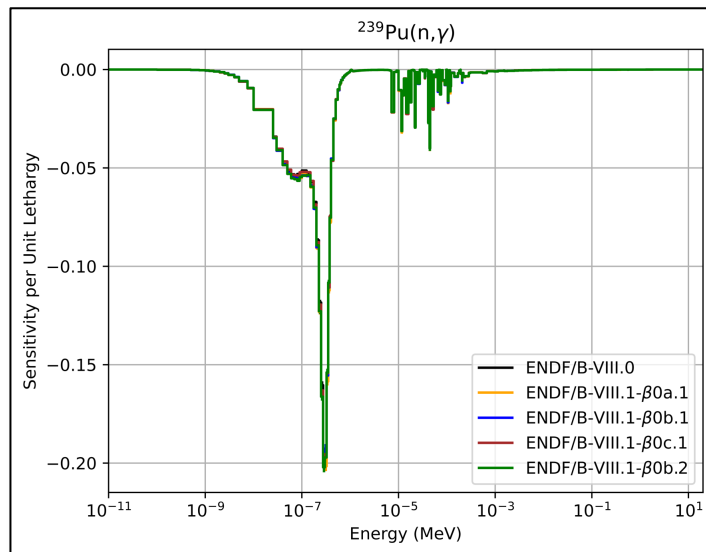
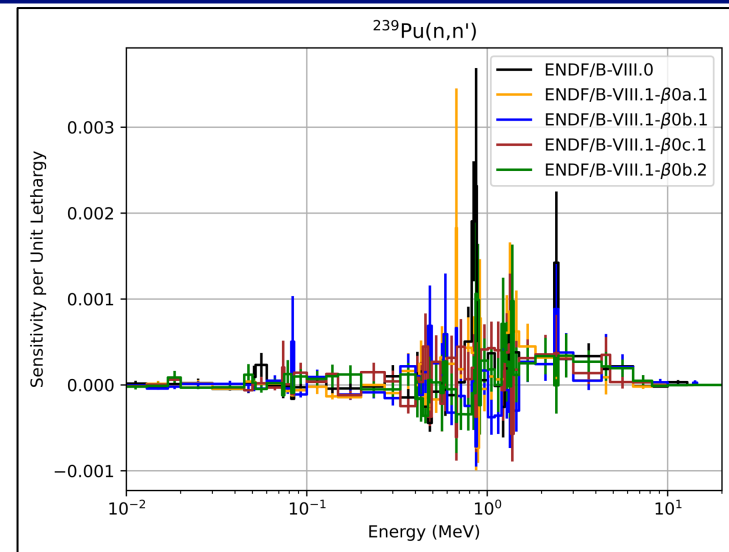
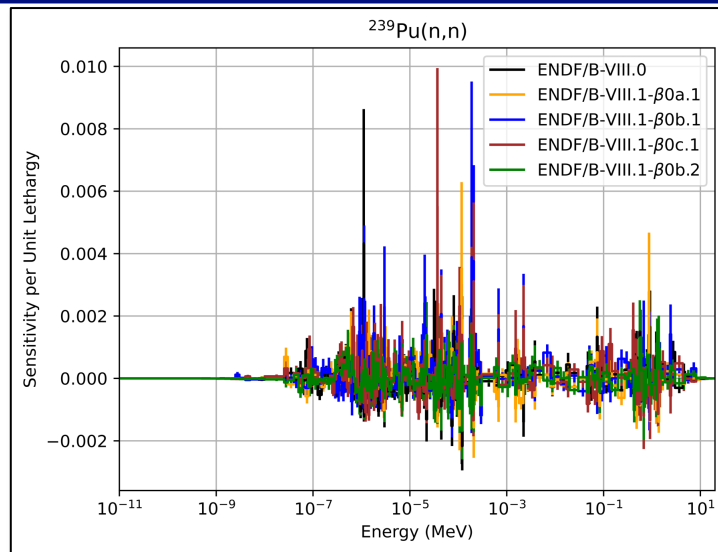
Results – ENDF/B-VIII.0 $S(\alpha, \beta)$, ENDF/B-VIII- $\beta 0$ nuclides, PU-MET-INTER/PU-SOL-THERM Benchmark Suite



Results – ENDF/B-VIII.0 $S(\alpha, \beta)$, ENDF/B-VIII- $\beta 0$ nuclides, PU-SOL-THERM-034 Case 2 Benchmark ^{239}Pu Sensitivity



Results – ENDF/B-VIII.0 $S(\alpha, \beta)$, ENDF/B-VIII- $\beta 0$ nuclides, PU-SOL-THERM-034 Case 2 Benchmark ^{239}Pu Sensitivity



Results – ENDF/B-VIII.0 $S(\alpha, \beta)$, ENDF/B-VIII- $\beta 0$ nuclides, Jezebel & Godiva Reaction Rate Ratios

	C/E				
Reaction Rate Ratio* <u>Jezebel</u>	ENDF/B-VIII.0	ENDF/B-VIII.1- $\beta 0$ a.1	ENDF/B-VIII.1- $\beta 0$ b.1	ENDF/B-VIII.1- $\beta 0$ c.1	ENDF/B-VIII.1- $\beta 0$ b.2
$^{169}\text{Tm}(n,2n)/^{235}\text{U}(n,f)$	1.18933	1.13941	1.12747	1.16556	<u>1.12708</u>
$^{191}\text{Ir}(n,2n)/^{235}\text{U}(n,f)$	1.14892	1.10078	1.08910	1.12578	<u>1.08870</u>
$^{238}\text{U}(n,2n)/^{235}\text{U}(n,f)$	1.24899	1.20824	<u>1.20027</u>	1.23182	1.20248
$^{238}\text{U}(n,f)/^{235}\text{U}(n,f)$	0.99979	0.98351	0.99330	<u>1.00007</u>	0.99440
$^{239}\text{Pu}(n,f)/^{235}\text{U}(n,f)$	0.99817	0.99513	0.99599	<u>0.99830</u>	0.99624
Reaction Rate Ratio <u>Godiva</u>					
$^{239}\text{Pu}(n,f)/^{235}\text{U}(n,f)$	1.01800	<u>1.01685</u>	1.01714	1.01702	1.01714

*Reaction rate ratio measured values taken from Reference 3

Results – ENDF/B-VIII.0 $S(\alpha, \beta)$, ENDF/B-VIII- $\beta 0$ nuclides, Flattop-Pu & Flattop-25 Reaction Rate Ratios

	C/E				
Reaction Rate Ratio Flattop-Pu	ENDF/B-VIII.0	ENDF/B-VIII.1- $\beta 0$ a.1	ENDF/B-VIII.1- $\beta 0$ b.1	ENDF/B-VIII.1- $\beta 0$ c.1	ENDF/B-VIII.1- $\beta 0$ b.2
$^{169}\text{Tm}(n,2n)/^{235}\text{U}(n,f)$	1.25283	1.20937	1.21469	1.23000	<u>1.19918</u>
$^{191}\text{Ir}(n,2n)/^{235}\text{U}(n,f)$	1.06656	1.03015	1.03437	1.04733	<u>1.02093</u>
$^{203}\text{Tl}(n,2n)/^{235}\text{U}(n,f)$	1.28912	1.24569	1.24887	1.26543	<u>1.23412</u>
$^{238}\text{U}(n,2n)/^{235}\text{U}(n,f)$	1.20523	1.17612	1.17587	1.19383	<u>1.17270</u>
$^{238}\text{U}(n,f)/^{235}\text{U}(n,f)$	1.04771	<u>1.03017</u>	1.03914	1.04840	1.04086
$^{239}\text{Pu}(n,f)/^{235}\text{U}(n,f)$	0.97694	0.97403	0.97468	<u>0.97719</u>	0.97493
Reaction Rate Ratio Flattop-25					
$^{239}\text{Pu}(n,f)/^{235}\text{U}(n,f)$	<u>0.98522</u>	0.98396	0.98428	0.98416	0.98428

Results – ENDF/B-VIII.0 $S(\alpha, \beta)$, ENDF/B-VIII- $\beta 0$ nuclides, Neutron Reactor Kinetics Parameters

Metric*	Benchmark	C/E				
		ENDF/B-VIII.0	ENDF/B-VIII.1- $\beta 0a.1$	ENDF/B-VIII.1- $\beta 0b.1$	ENDF/B-VIII.1- $\beta 0c.1$	ENDF/B-VIII.1- $\beta 0b.2$
Prompt Neutron Decay Constant α	Jezebel	1.01591	<u>0.99934</u>	1.00420	1.02411	1.01617
	Flatop-Pu	<u>1.00035</u>	0.98457	0.98773	1.00533	0.99579
	Godiva	1.04467	<u>1.03323</u>	<u>1.03323</u>	<u>1.03323</u>	<u>1.03323</u>
	Flatop-25	1.05728	<u>1.04678</u>	<u>1.04678</u>	<u>1.04678</u>	<u>1.04678</u>
Effective Delayed Neutron Fraction β_{eff}	Jezebel	0.94845	0.94845	0.94845	0.95361	<u>0.95876</u>
	Godiva	<u>0.98786</u>	0.97724	0.97724	0.97724	0.97724

*Measured neutron reactor kinetics parameter values taken from Reference 4 and Reference 5

Conclusions

- A lot of results to digest...
- Benchmarks that don't include plutonium aren't changing as much, or in the direction we expect them to
- Plutonium benchmark results show a lot of promise – each Pu-239 file has qualities that make it desirable according to specific validation calculations – it would be beneficial to use the results shown in this presentation to design collaborative pathways forward
- Additional validation results are required to examine smaller trends for more conclusive statements

Acknowledgments

The validation work shown in these slides would not have been possible without the support and mentorship from Denise Neudecker and Skip Kahler. Skip Kahler not only produced some of the results shown in this presentation, but also helped to verify results. I would also like to acknowledge the hard work Nathan Gibson put into processing the ENDF-formatted files into ACE-formatted files that were used for all validation results shown and Wim Haeck for double checking all the produced ACE-formatted files.

The work performed at Los Alamos National Laboratory for this presentation was carried out under the auspices of the NNSA of the United States Department of Energy under contract 89233218CNA000001.

References

1. J. D. BESS, T. IVANOVA, L. SCOTT, I. HILL, “The 2019 Edition of the ICSBEP Handbook,” *Transactions of the American Nuclear Society*, **121**, 901-904 (2019).
2. J. A. Kulesza et al., “MCNP® Code Version 6.3.0 Theory & User Manual,” Los Alamos National Laboratory Tech. Rep. LA-UR-22-30006, Rev. 1.
3. M. B. Chadwick et al., “The CIELO Collaboration: Neutron Reactions on ^1H , ^{16}O , ^{56}Fe , $^{235,238}\text{U}$, and ^{239}Pu ,” *Nuclear Data Sheets*, **118**, 1-25 (2014).
4. D. A. Brown et al., “ENDF/B-VIII.0: The 8th Major Release of the Nuclear Reaction Data Library with CIELO-project Cross Sections, New Standards and Thermal Scattering Data,” *Nuclear Data Sheets*, **148**, 85 (2018).
5. G. Mckenzie, “Validation of MCNP® Rossi-Alpha Calculations Using Recent Measurements,” *Proceeding of the 11th International Conference on Nuclear Criticality Safety* (2019), Paris, France.

Validation of ENDF/B-VIII.1- $\beta 0$ -based Continuous Energy Data Tables

Noah Kleedtke*, Skip Kahler, Wim Haeck, Denise Neudecker,
Nathan Gibson

XCP-5: Materials and Physical Data
Los Alamos National Laboratory

2022 CSEWG Annual Meeting – Nuclear Data Validation

LA-UR-22-31596

*kleedtke@lanl.gov