



January 8, 2026

Impact of cross section and fission yield uncertainties on the fuel inventory in a high temperature fluoride salt-cooled reactor

Nuclear Data Week 2025, CSEWG

F. Bostelmann, R. Elzohery, G. Procop

Oak Ridge National Laboratory



U.S. DEPARTMENT
of **ENERGY**

ORNL IS MANAGED BY UT-BATTELLE LLC
FOR THE US DEPARTMENT OF ENERGY



Motivation

Advanced reactors

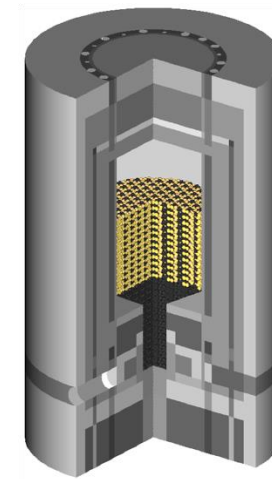
- Gained interest to meet global energy demands
- Increase in deployment expected in the near to mid future
- Significantly different from well-studied LWRs: different fuel forms, coolants, enrichments (e.g., HALEU), elevated burnups and temperatures

Nuclear data

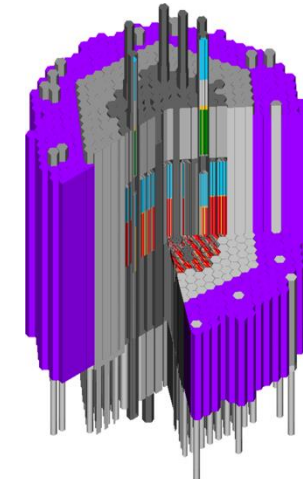
- Limited experimental data increases reliance on mod&sim
- Impact of uncertainties in nuclear data on mod&sim must be assessed
 - Reactivity: Several assessments were conducted
 - Fuel inventory: Very few studies available

This work

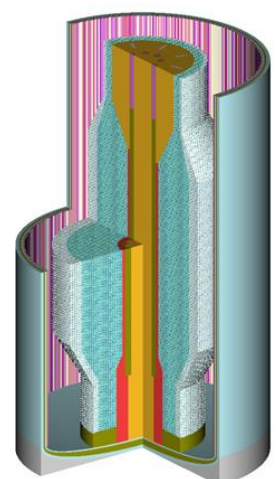
- Propagate cross section and fission yield uncertainties to the fuel inventory during operation and spent fuel inventory of a fluoride salt-cooled high-temperature pebble-bed reactor (pebble-bed FHR)
- Identify key contributors to resulting uncertainties



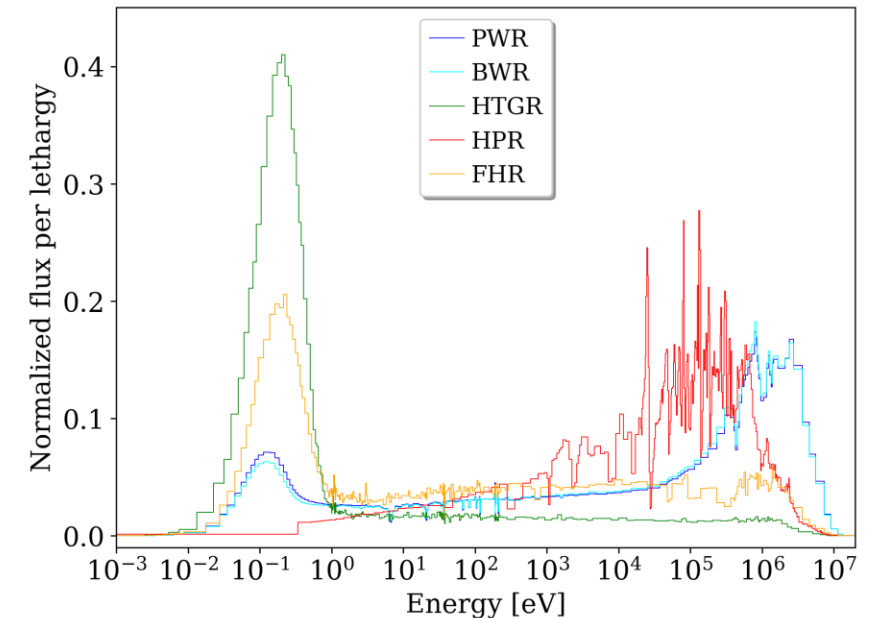
Pebble-bed HTGR



SFR



Pebble-bed FHR



Neutron spectrum: advanced reactors vs. LWRs

Application: UC Berkeley FHR Mark I

General characteristics:

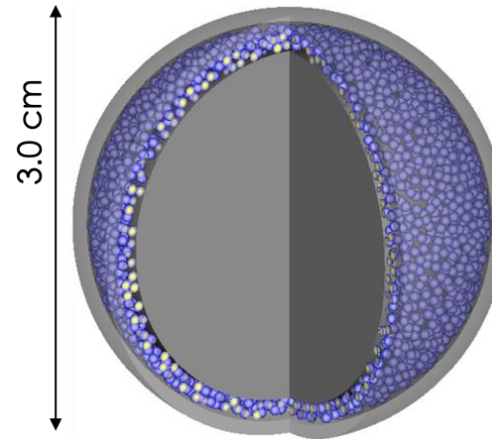
- 236 MWth
- 470,000 fuel pebbles + 218,000 graphite pebbles
- **FLiBe** molten salt coolant, $T \approx 873\text{--}973\text{ K}$
- **Graphite** reflector, $T \approx 873\text{--}973\text{ K}$

Fuel pebbles:

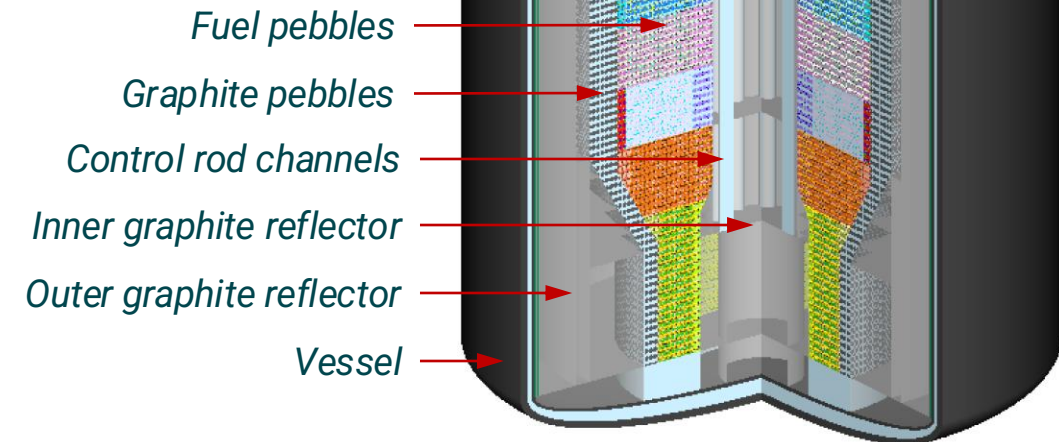
- TRISO fuel particles at 40% packing fraction
- **UCO fuel with 19.9 wt% ^{235}U enrichment**, $T \approx 1,003\text{ K}$
- Average discharge burnup: **180 GWd/tU**

Equilibrium core:

- Fuel pebbles at target burnup continuously replaced by fresh fuel pebbles
- Core contains fuel pebbles at different levels of burnup
- Average conditions in each zone approximately constant: average inventory / burnup, flux and power profiles, temperatures, etc.



FHR fuel pebble
(4,730 TRISOs)



FHR core

Approach

1. Spent and equilibrium core fuel inventory:

- Application of "SCALE Leap-In method for Cores at Equilibrium" (SLICE)
- Depletion of fuel pebbles in representative spectral environment
- Tool: SCALE/TRITON with KENO-VI and ORIGEN

2. Uncertainty and sensitivity analysis:

- Generate random samples of XS and FPY using covariance data
- Perform 1,000 calculations using these random samples
- Perform statistical analysis of results
- Tool: SCALE/Sampler
 - Importance ranking of cross sections in terms of sensitivity index $R^2 \in [0,1]$

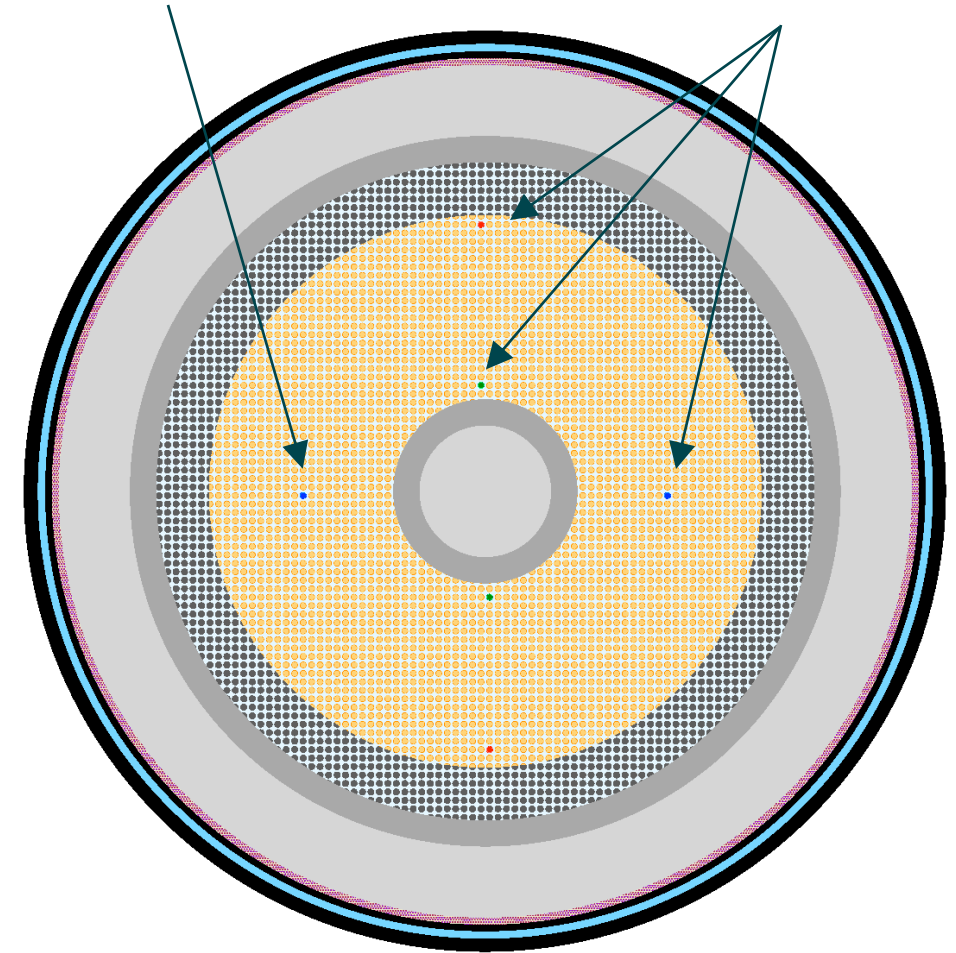
Covariance library: ENDF/B-VII.1 or ENDF/B-VIII.1, plus BNL-LANL-ORNL (BLO) uncertainty data

Core-average fuel pebbles:

- Core-average fuel composition
- **Not depleted**

Fresh fuel pebbles:

- Fresh fuel composition
- **Depleted**



Axial slice through the FHR core

Approach

Calculations performed with the SCALE code system and ENDF/B-VII.1 or ENDF/B-VIII.1 data, plus BNL-LANL-ORNL (BLO) uncertainty data

1. Spent and equilibrium core fuel inventory:

- TRITON reactor physics sequence
- KENO-VI Monte Carlo neutron transport
- ORIGEN depletion and decay solver

2. Uncertainty and sensitivity analysis:

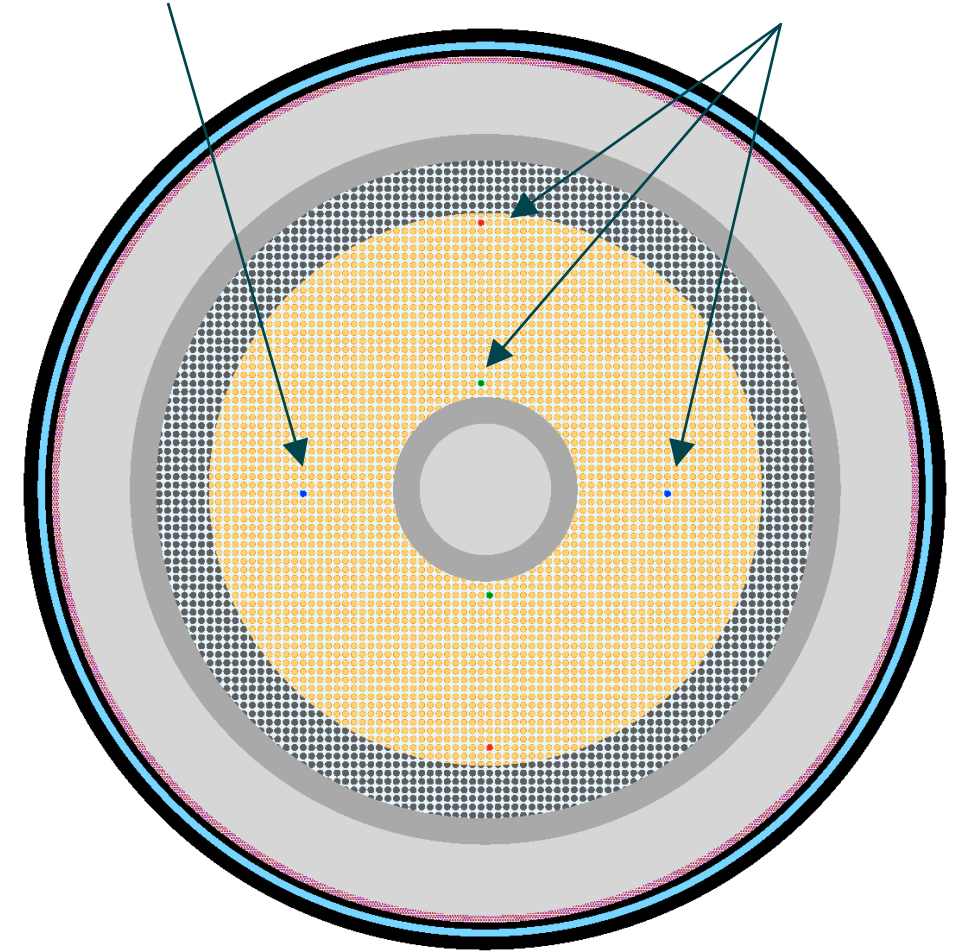
- Sampler uncertainty and sensitivity analysis sequence
- Importance ranking of cross sections in terms of sensitivity index $R^2 \in [0,1]$

Core-average fuel pebbles:

- Core-average fuel composition
- Not depleted

Fresh fuel pebbles:

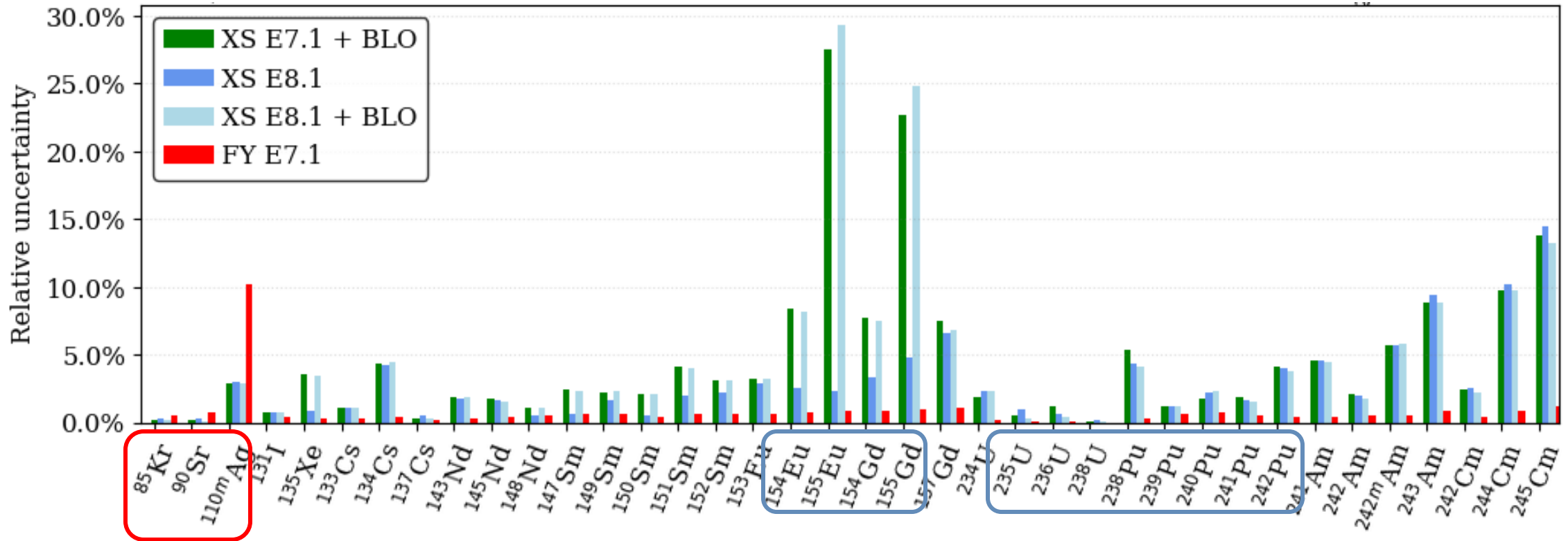
- Fresh fuel composition
- Depleted



Axial slice through the FHR core

Relative uncertainty of spent FHR fuel inventory at discharge

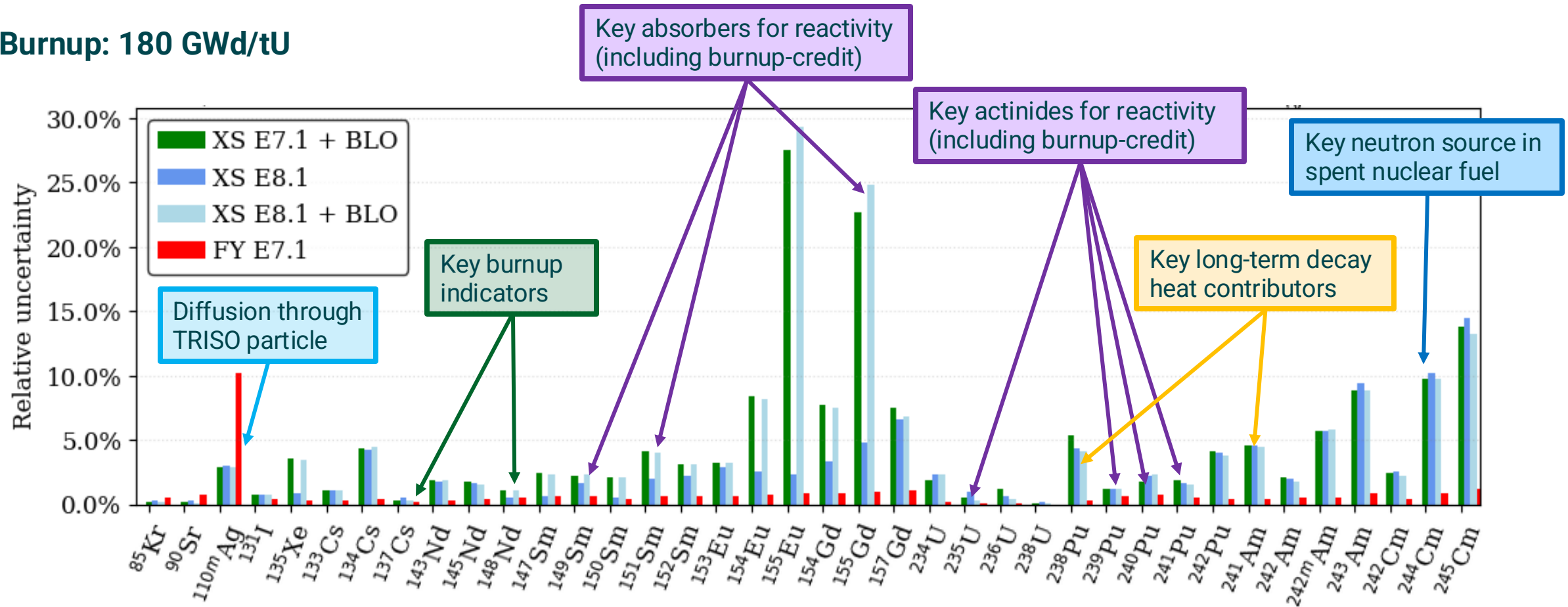
Burnup: 180 GWd/tU



Cross section uncertainties are major drivers for uncertainties, with few exceptions for $\text{Ag}^{110\text{m}}$, Kr^{85} , Sr^{90}
Differences between E8.1+BLO and E7.1+BLO are small, mostly below <1% relative difference
Impact of BLO uncertainties for key fission products Sm, Eu, Gd point to important missing covariance data

FHR fuel inventory uncertainties are relevant for different applications

Burnup: 180 GWd/tU



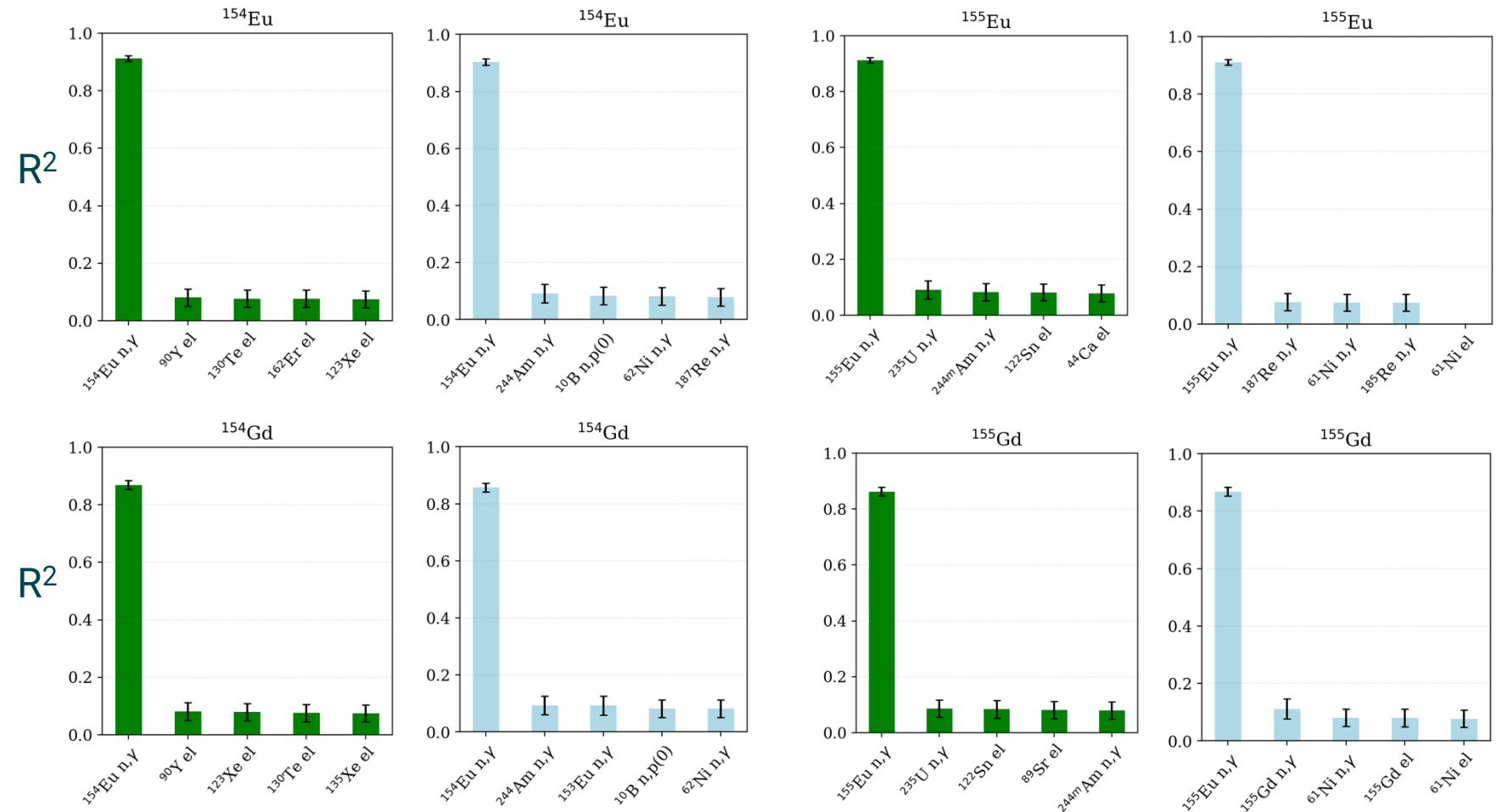
Cross section uncertainties are major drivers for uncertainties, with few exceptions for Ag^{110m} , Kr^{85} , Sr^{90}
Differences between E8.1+BLO and E7.1+BLO are small, mostly below <1% relative difference
Impact of BLO uncertainties for key fission products Sm, Eu, Gd point to important missing covariance data

Sensitivity analysis reveals missing covariance data influencing important neutron absorber

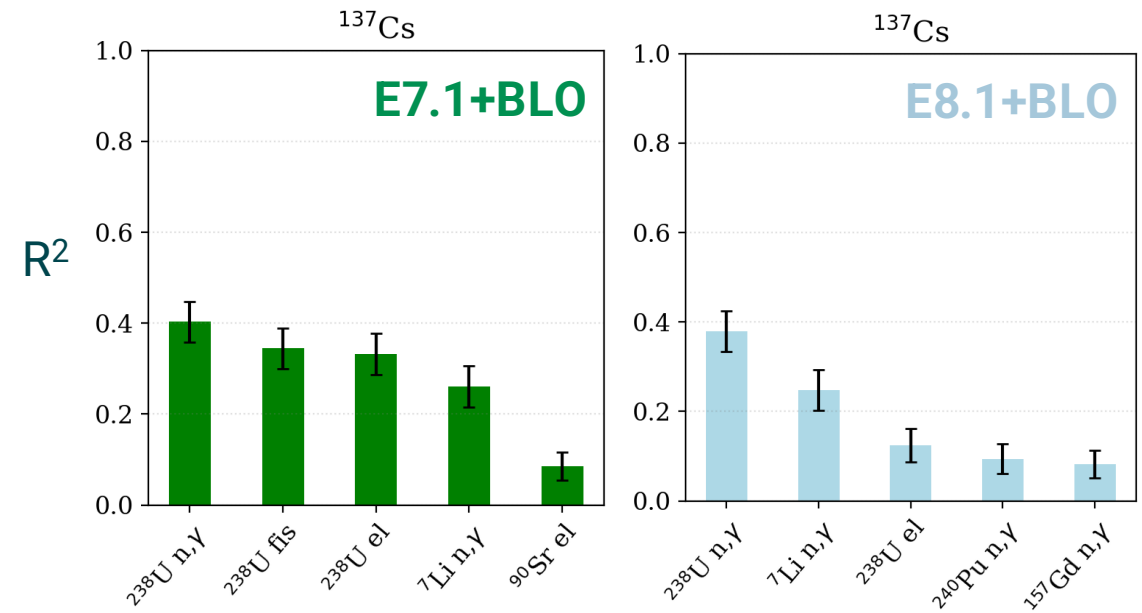
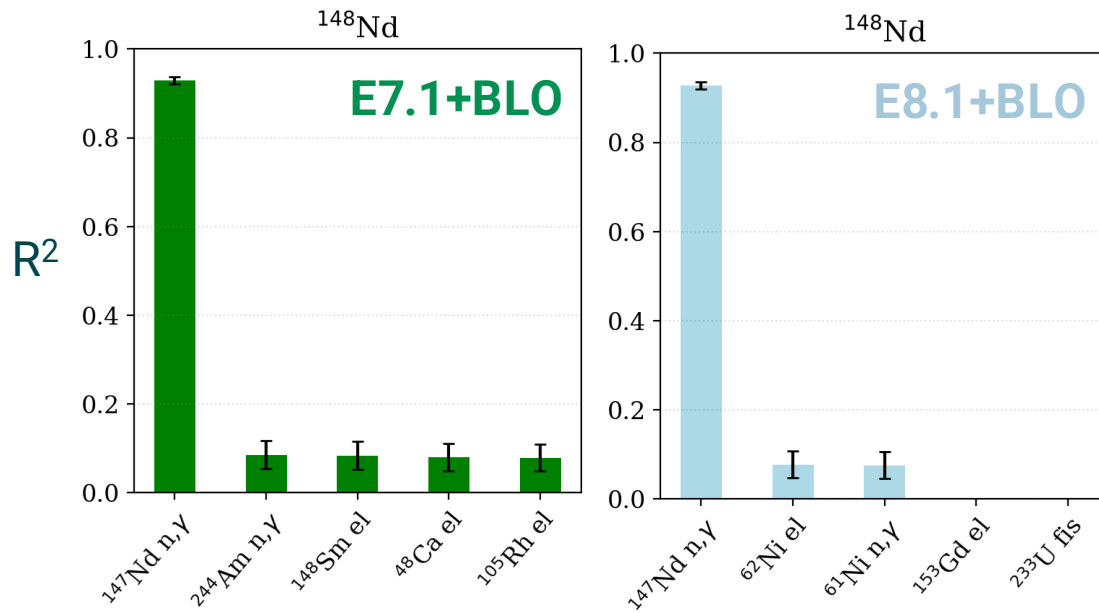
E7.1+BLO

E8.1+BLO

- Uncertainty in Eu-154 (n,γ) is the main contributor to Eu-154 and Gd-154 concentration uncertainty
- Uncertainty in Eu-155 (n,γ) is the main contributor to Eu-155 and Gd-155 concentration uncertainty
- **BUT: Eu-154 (n,γ) and Eu-155 (n,γ) covariance data is not included in E8.1, only in BLO**

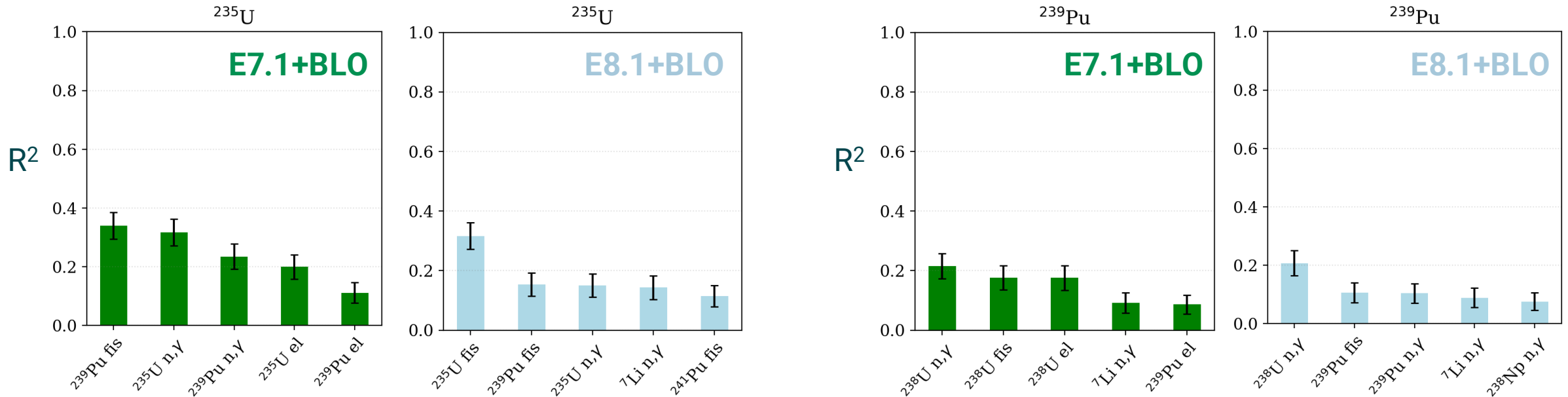


Sensitivity analysis reveals missing and updated covariance data impacting burnup indicators



- Uncertainty in Nd-148 concentration is mainly impacted by Nd-147 (n, γ)
BUT: Nd-147 (n, γ) covariance data is not included in E8.1, only in BLO
- Uncertainty in Cs-137 concentration (0.3%-0.5%) showed small changes between libraries, mainly due to updates in U-238 (n, γ) and fission

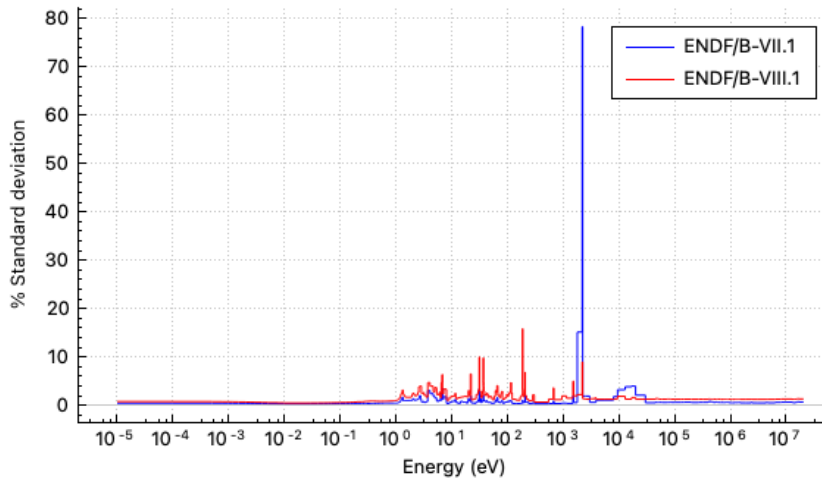
Sensitivity analysis reveals covariance data updates impacting major actinides U-235 and Pu-239



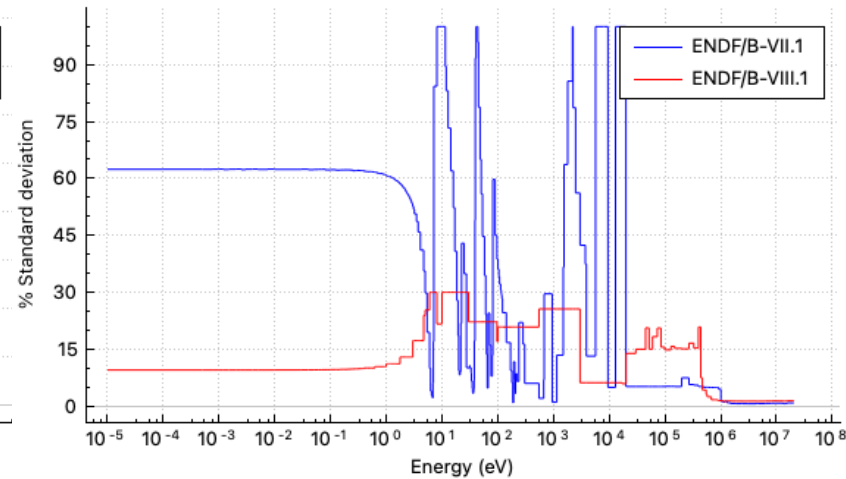
- Uncertainty in U-235 concentration (0.4%-0.5%) shows only small change between libraries, likely due to compensating effects from updates of (n, γ) and fission in U-235 and Pu-239
- Uncertainty in Pu-239 concentration (1.2%) did not change between libraries; but individual uncertainty contributions changed, mainly due to updates in U-238 (n, γ) and fission

Relevant cross section uncertainty updates between E7.1 and E8.1

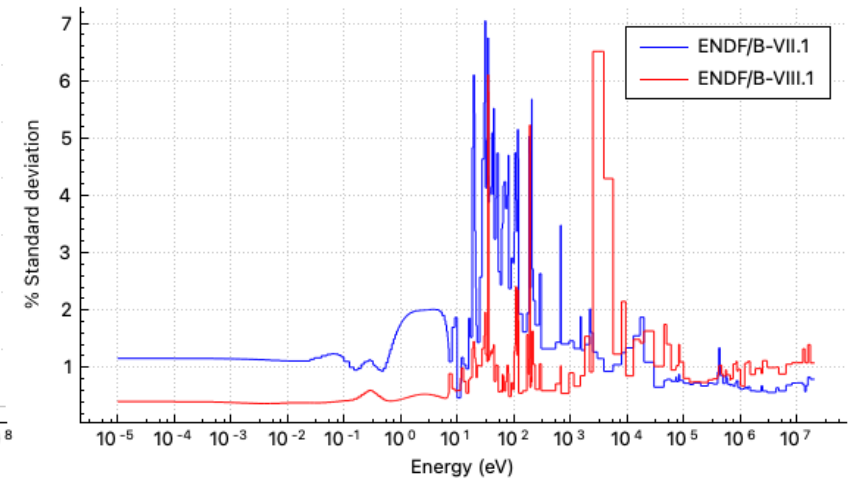
U-235 mt=18 fission



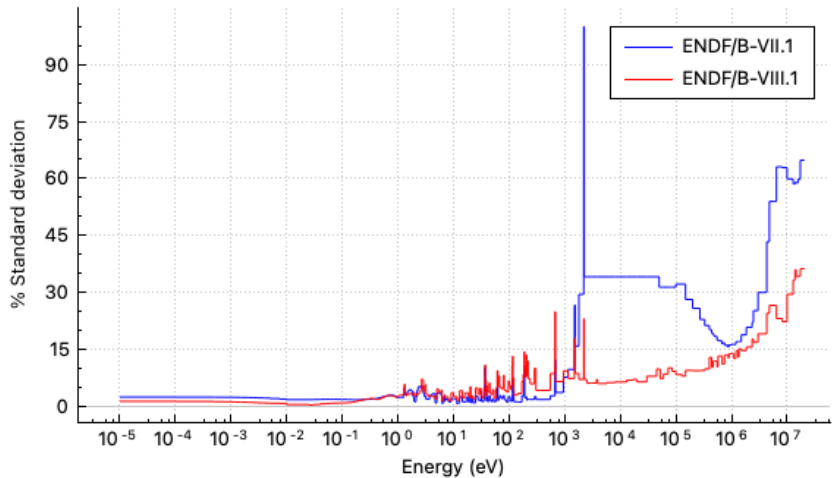
U-238 mt=18 fission



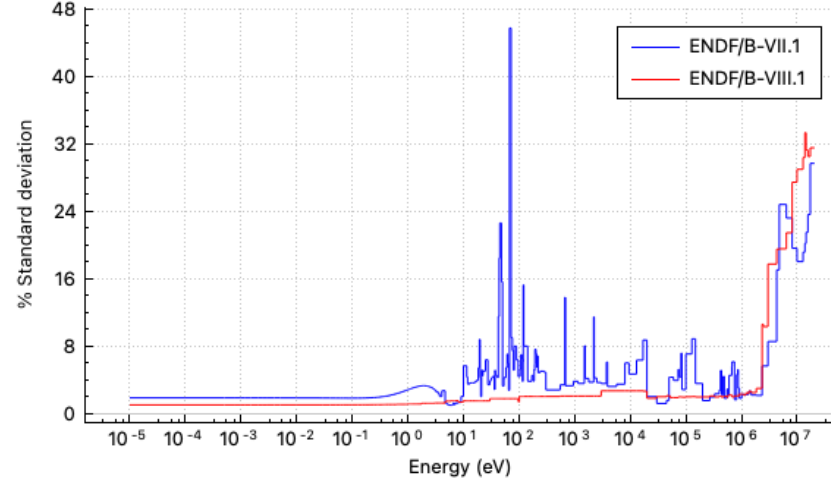
Pu-239 mt=18 fission



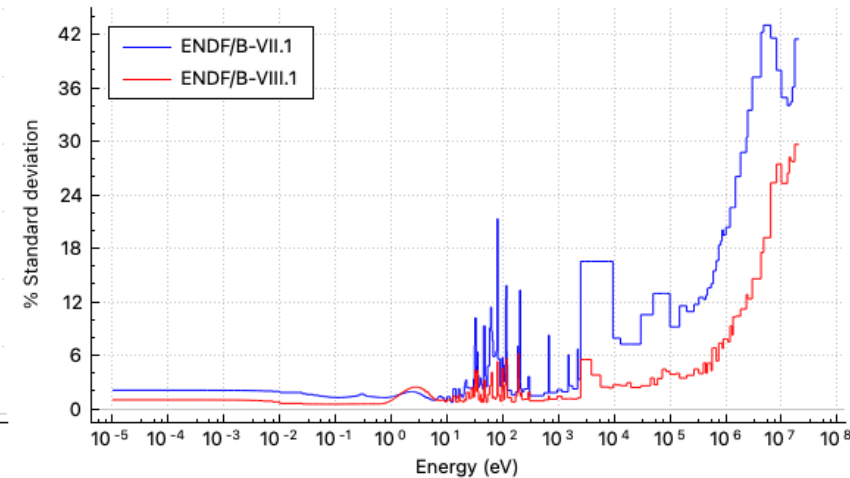
U-235 mt=102 n,gamma



U-238 mt=102 n,gamma



Pu-239 mt=102 n,gamma



Summary and Conclusions

Random sampling approach was applied to investigate the impact of uncertainties in cross sections and fission yields on uncertainties in FHR spent fuel pebble inventory

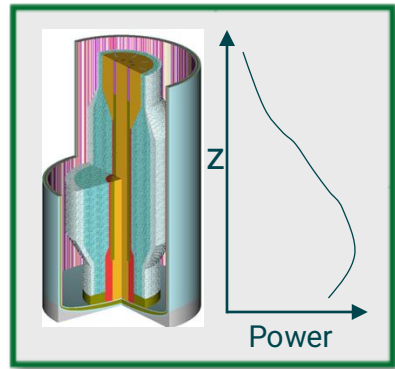
- Fission product yield uncertainties have minor impact, with few exceptions
- Cross section uncertainties cause inventory uncertainties of up to 25% (Sm, Eu, Gd)
- Comparison of cross section uncertainties between libraries highlight:
 - Impact of covariance data update in ENDF/B-VIII.1: U-235, U-238, and Pu-239 (n, γ) and fission
 - Missing important covariance data ENDF/B-VIII.1, as identified through use of BLO data: Nd-148 (n, γ) and Eu-155 (n, γ)
- *Preliminary results presented, with the more comprehensive analysis to follow soon*

Sensitivity analysis identified cross sections that are top contributors of uncertainties in fuel inventory predictions:

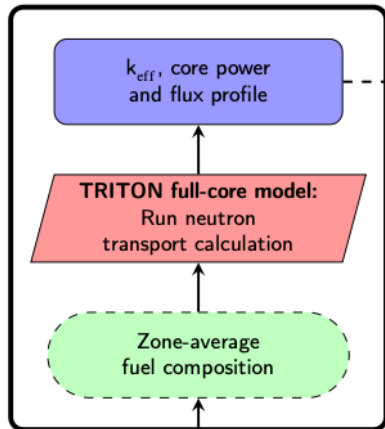
- Change in cross section importance points to relevant cross section uncertainty updates between ENDF/B libraries
- Use of BLO-data reveals impact of cross section covariance data missing in ENDF/B-VIII.1
- Identified top contributing cross sections can inform recommendations for new measurements and evaluations

Backup

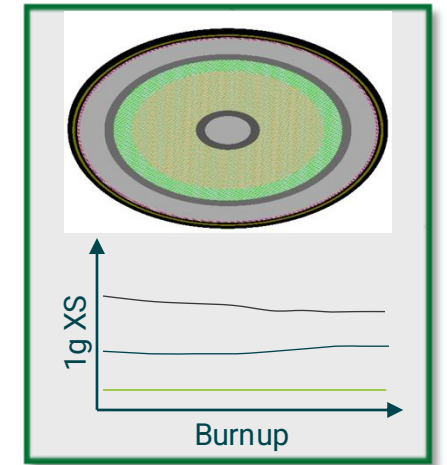
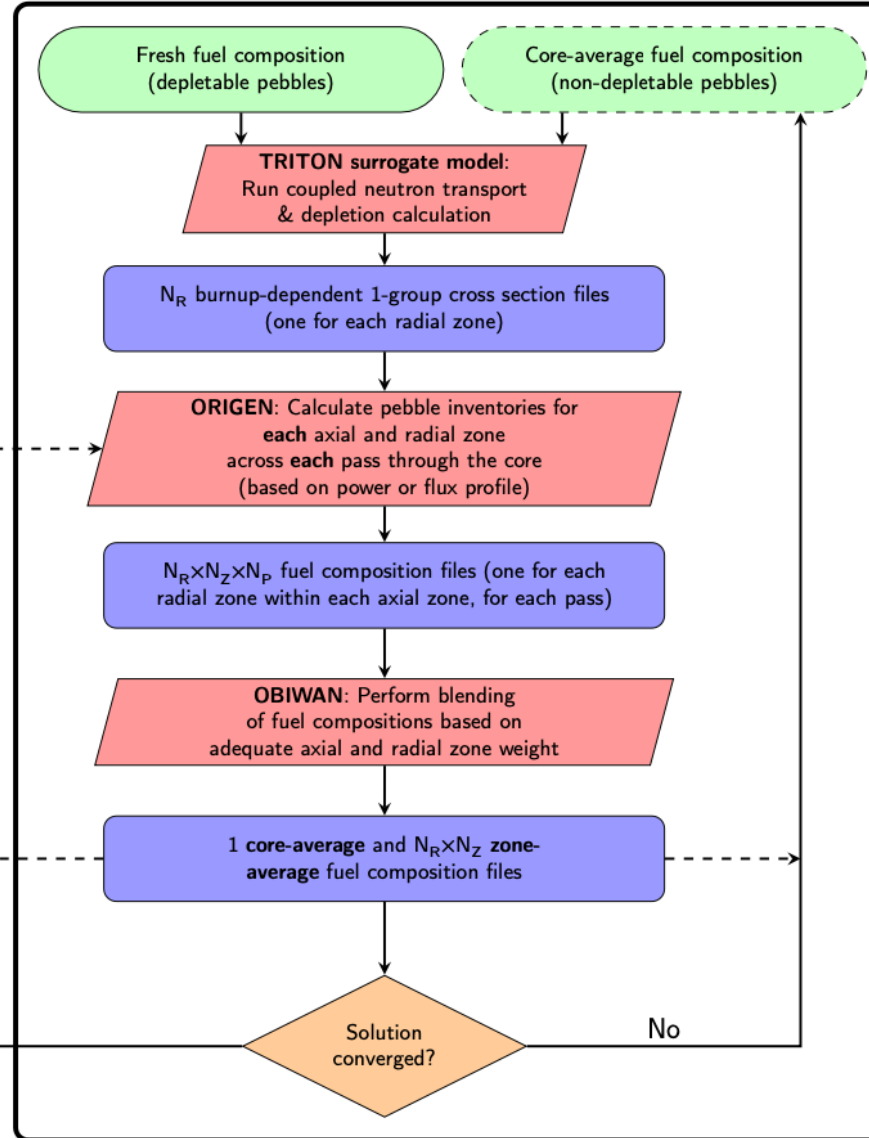
SLICE method for PBR equilibrium core generation



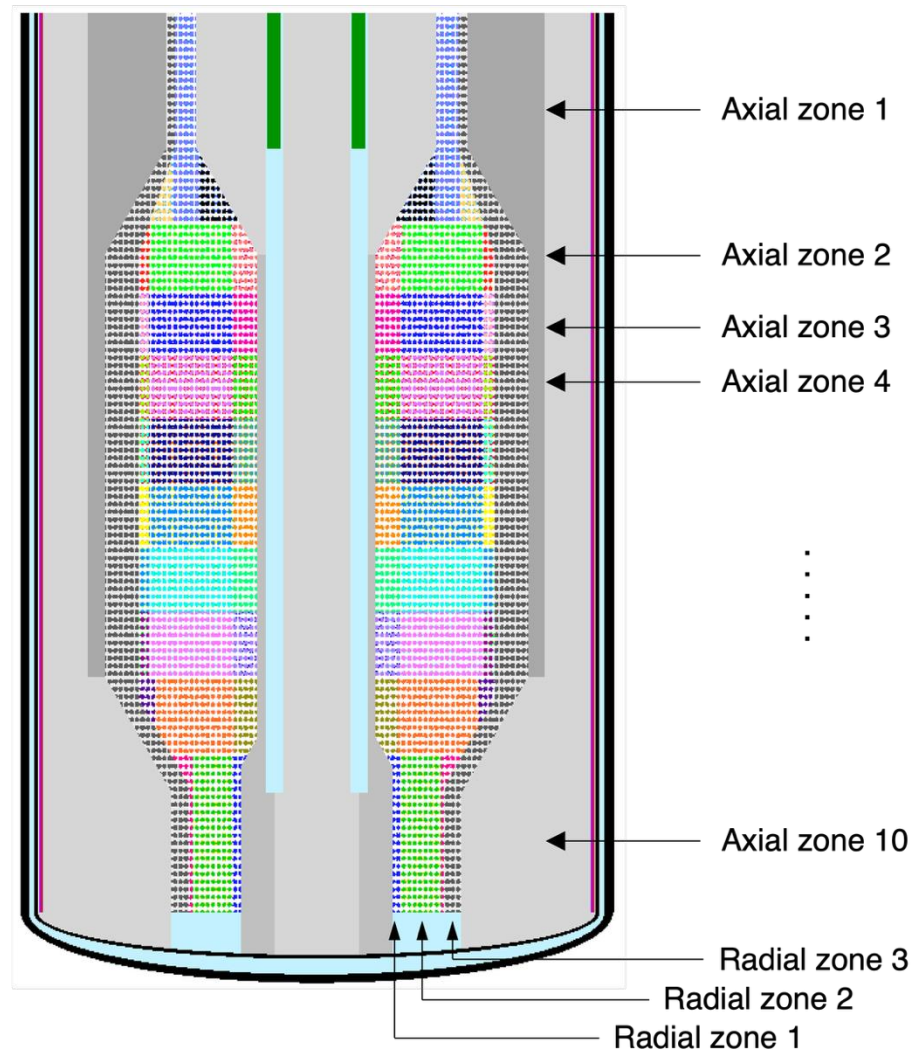
Outer iteration:
Calculate k_{eff} , power/flux profiles



Inner iteration:
Calculate zone-average fuel compositions



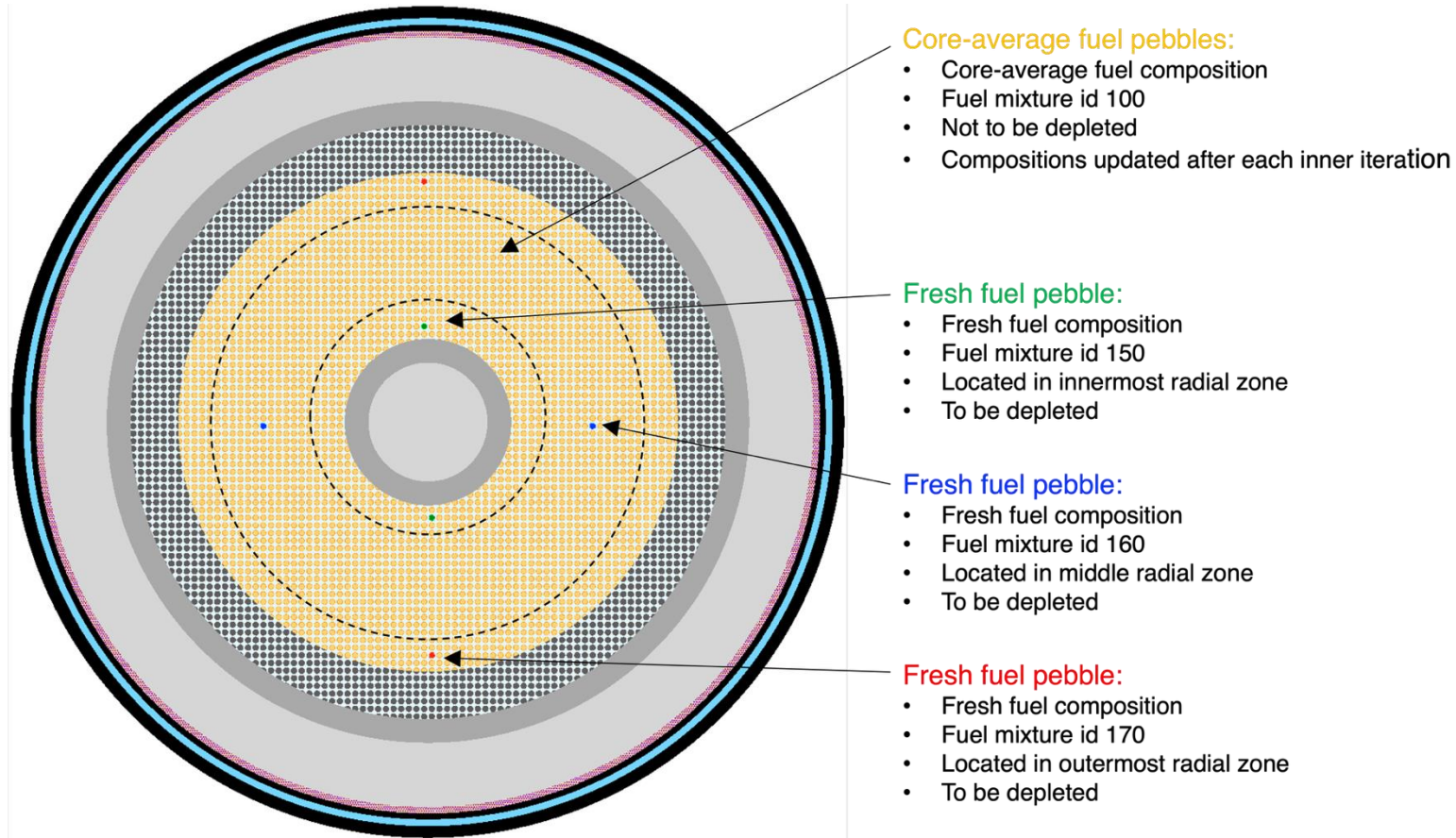
Discretized full core model



- Model discretization based on radial and axial flux profile
- Each region (i.e. specific radial zone within specific radial zone) has unique region-average fuel inventory
- Region-average fuel inventory represents average of fuel pebbles at different passes, at different burnups in this region
- TRITON calculation of this model results in axial/radial flux/power profiles

**Example FHR full core with
discretization of fuel region**

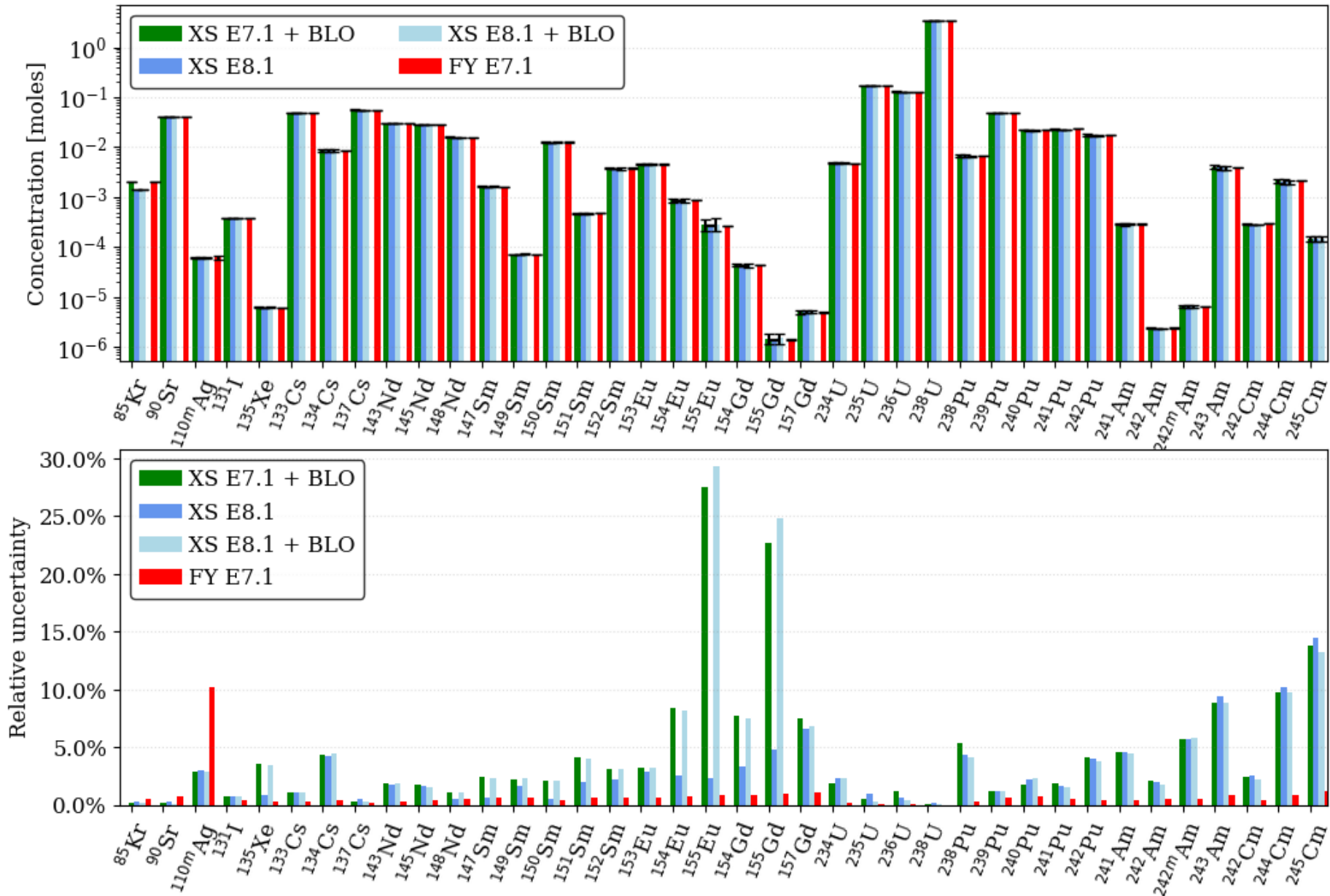
Axial slice surrogate model



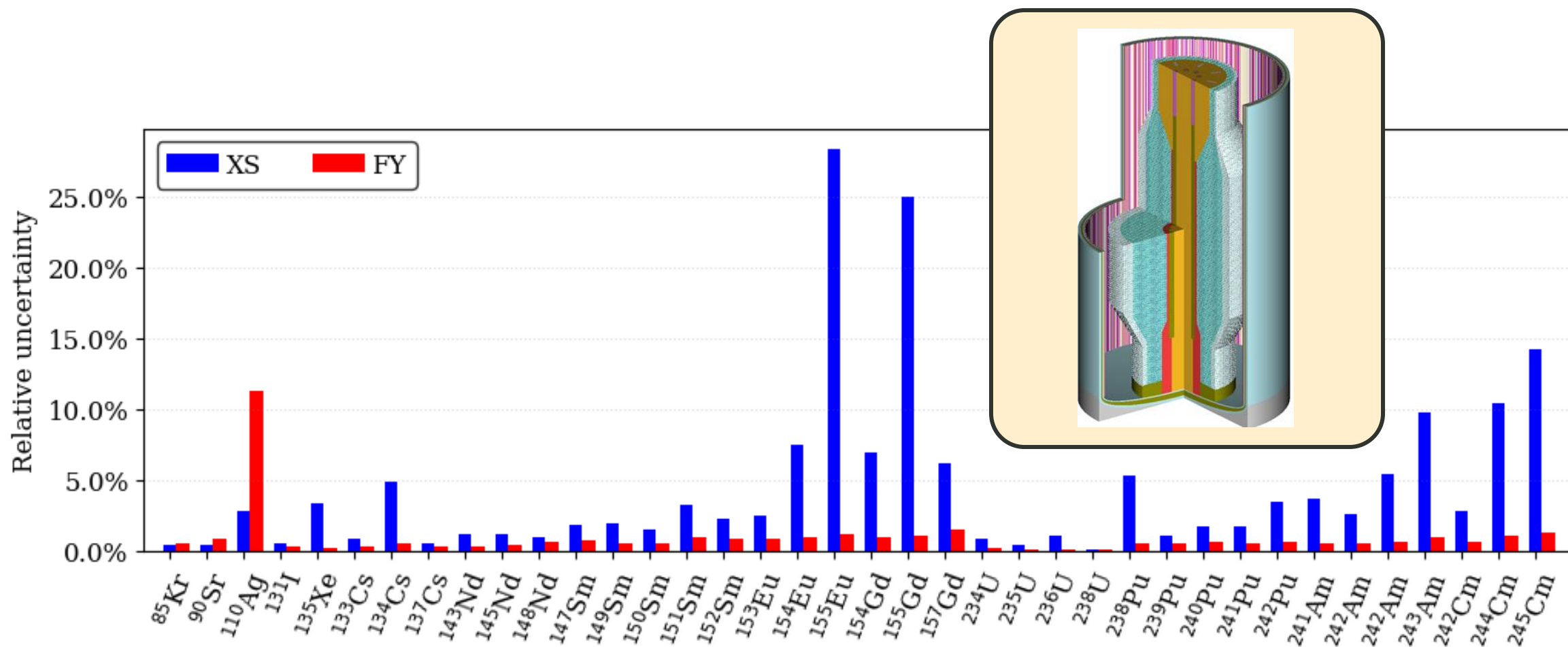
- TRITON model of axial slice through center of the core
- Depletion of only “depletable” fuel pebbles to target discharge burnup
- “Core-average” fuel pebbles provide representative spectral conditions
- TRITON depletion calculation of this model results in ORIGEN cross section libraries (1-group cross sections)

Example FHR axial slice model

Inventory of a discharged FHR fuel pebble

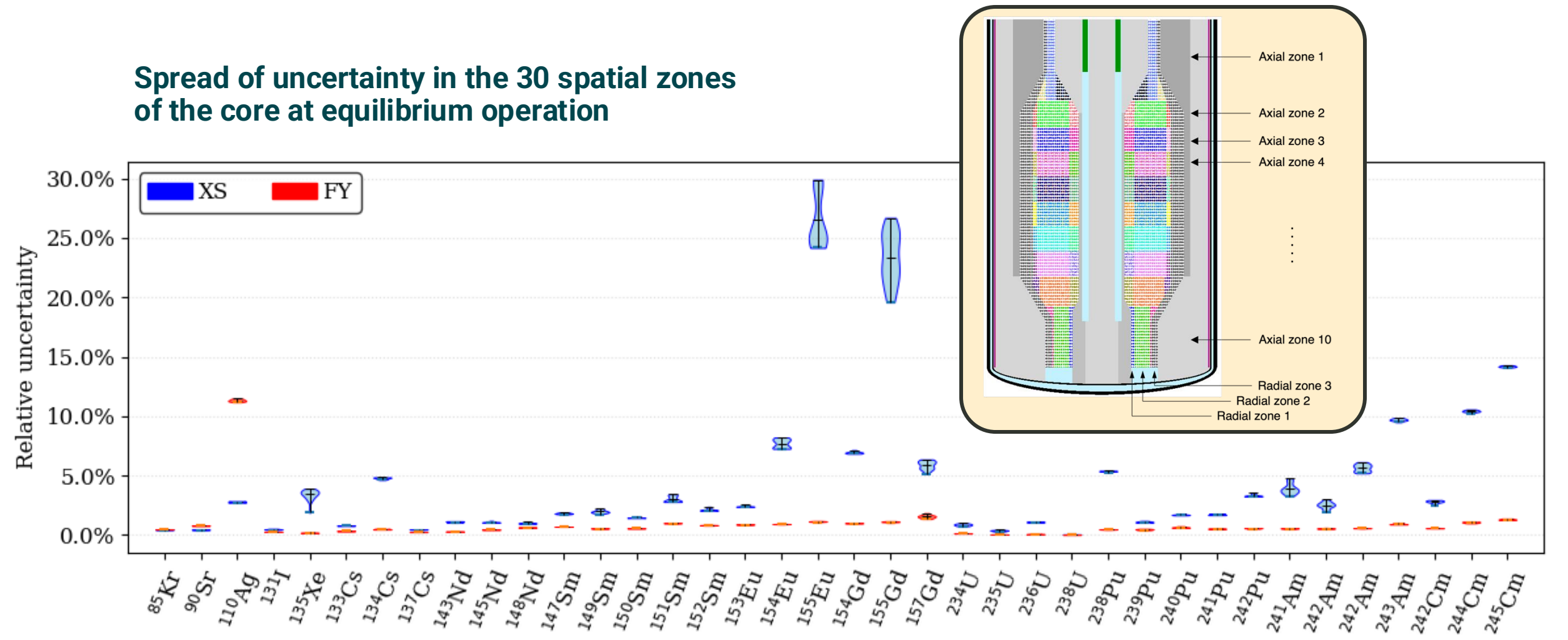


Core-average FHR inventory uncertainty at equilibrium operation using ENDF/B-VII.1



Uncertainties are approximately the same as for the spent fuel inventory

Zone inventory uncertainty at equilibrium operation using ENDF/B-VII.1



Significant uncertainty spread for only few nuclides: Eu, Gd, Cm isotopes