



January 8, 2026

Impact of cross section and fission yield uncertainties on the fuel inventory in a high temperature fluoride salt-cooled reactor

Nuclear Data Week 2025, CSEWG

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**U.S. DEPARTMENT
of ENERGY**

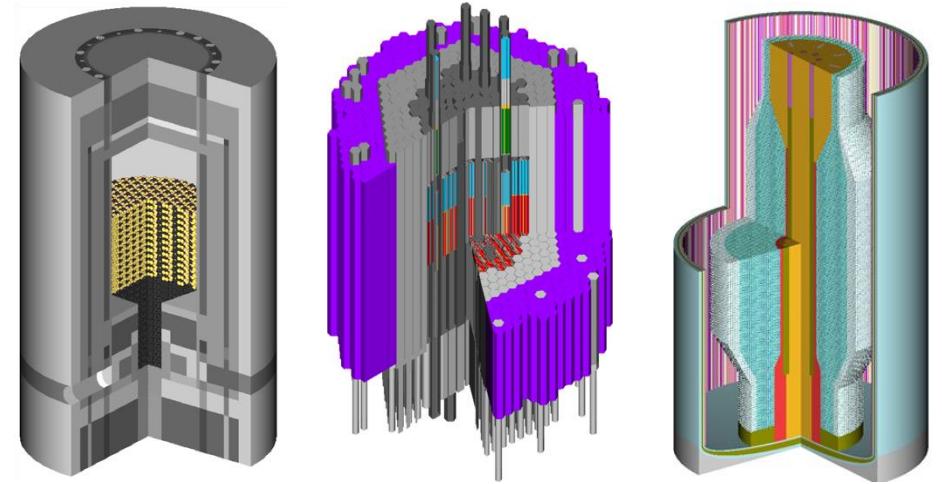
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Motivation

Advanced reactors

- Gained interest to meet global energy demands
- Increase in deployment expected in the near to mid future
- Significantly different from well-studied LWRs: different fuel forms, coolants, enrichments (e.g., HALEU), elevated burnups and temperatures



Pebble-bed HTGR

SFR

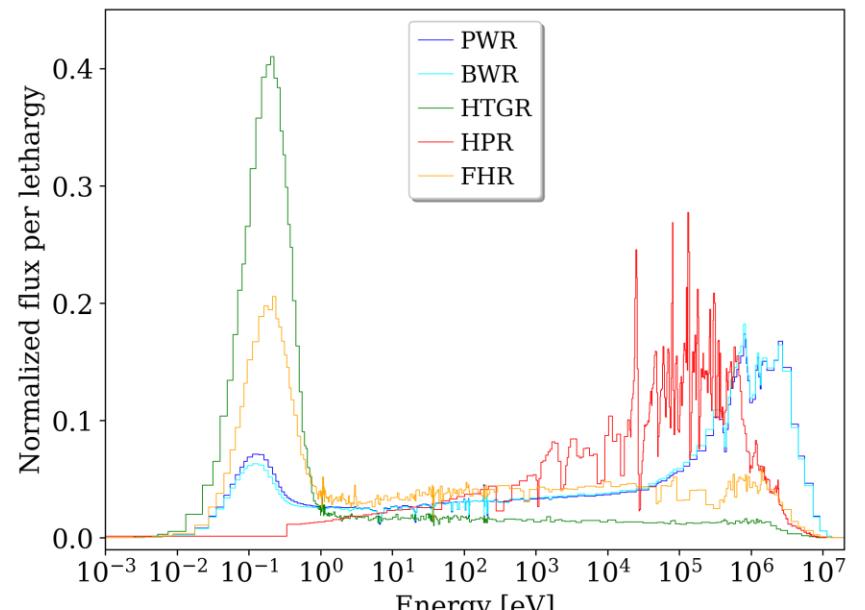
Pebble-bed FHR

Nuclear data

- Limited experimental data increases reliance on mod&sim
- Impact of uncertainties in nuclear data on mod&sim must be assessed
 - Reactivity: Several assessments were conducted
 - Fuel inventory: Very few studies available

This work

- Propagate cross section and fission yield uncertainties to the fuel inventory during operation and spent fuel inventory of a fluoride salt-cooled high-temperature pebble-bed reactor (pebble-bed FHR)
- Identify key contributors to resulting uncertainties



Neutron spectrum: advanced reactors vs. LWRs

Application: UC Berkeley FHR Mark I

General characteristics:

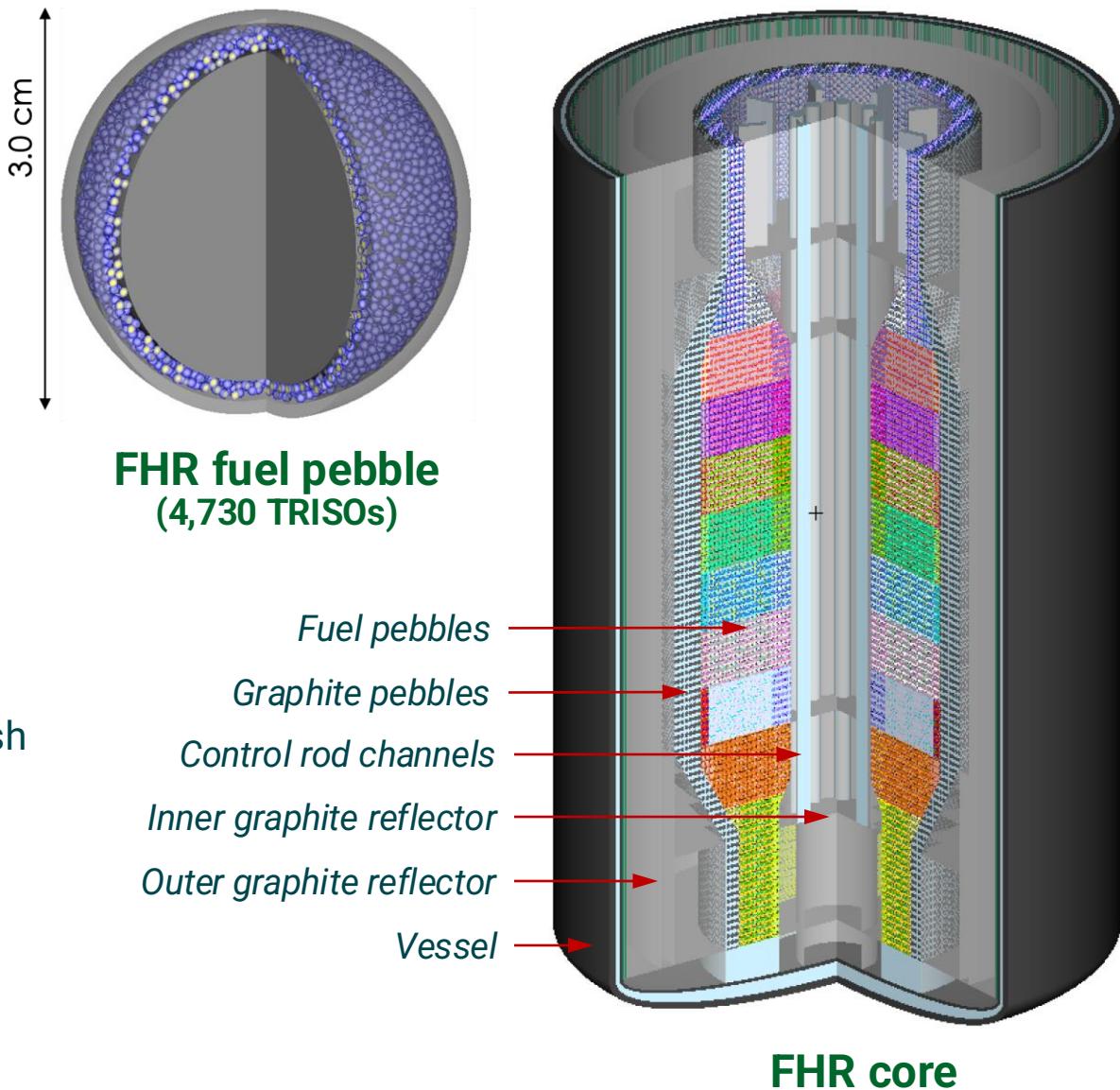
- 236 MWth
- 470,000 fuel pebbles + 218,000 graphite pebbles
- **FLiBe** molten salt coolant, $T \simeq 873\text{--}973\text{ K}$
- **Graphite** reflector, $T \simeq 873\text{--}973\text{ K}$

Fuel pebbles:

- TRISO fuel particles at 40% packing fraction
- **UCO fuel with 19.9 wt% ^{235}U enrichment**, $T \simeq 1,003\text{ K}$
- Average discharge burnup: **180 GWd/tU**

Equilibrium core:

- Fuel pebbles at target burnup continuously replaced by fresh fuel pebbles
- Core contains fuel pebbles at different levels of burnup
- Average conditions in each zone approximately constant: average inventory / burnup, flux and power profiles, temperatures, etc.



Approach

1. Spent and equilibrium core fuel inventory:

- Application of "SCALE Leap-In method for Cores at Equilibrium" (SLICE)
- Depletion of fuel pebbles in representative spectral environment
- Tool: SCALE/TRITON with KENO-VI and ORIGEN

2. Uncertainty and sensitivity analysis:

- Generate random samples of XS and FPY using covariance data
- Perform 1,000 calculations using these random samples
- Perform statistical analysis of results
- Tool: SCALE/Sampler
 - Importance ranking of cross sections in terms of sensitivity index $R^2 \in [0,1]$

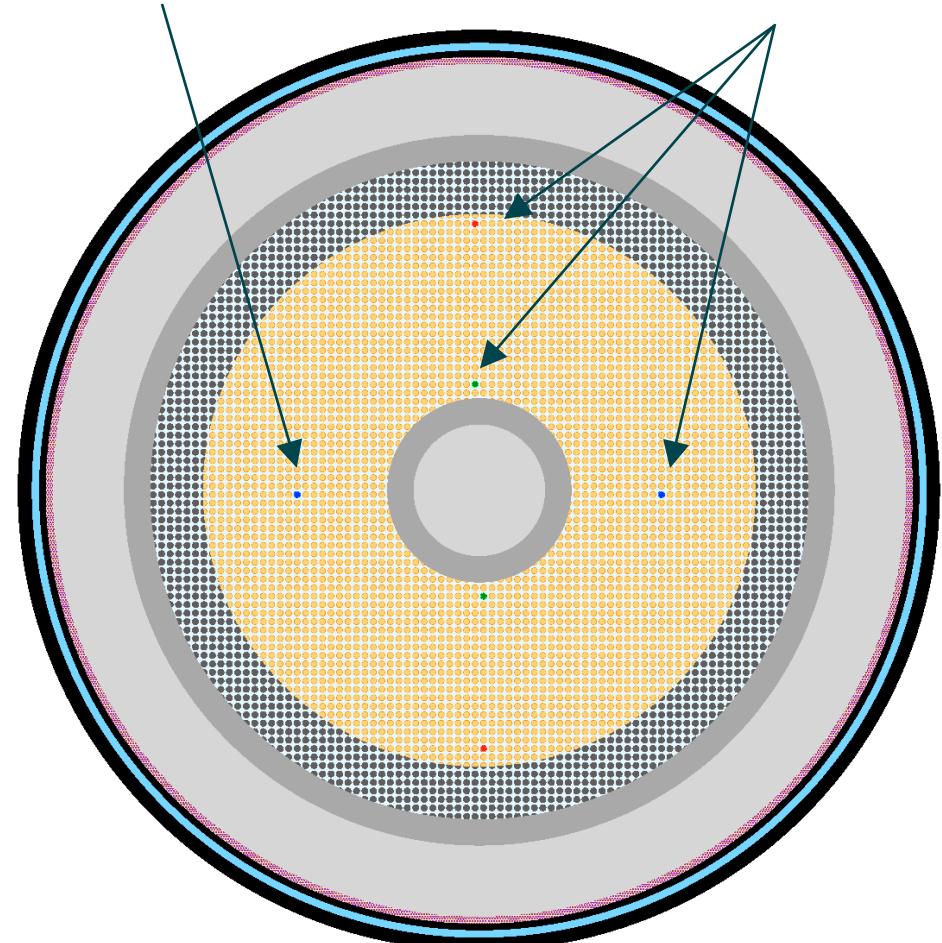
Covariance library: ENDF/B-VII.1 or ENDF/B-VIII.1,
plus BNL-LANL-ORNL (BLO) uncertainty data

Core-average fuel pebbles:

- Core-average fuel composition
- **Not depleted**

Fresh fuel pebbles:

- Fresh fuel composition
- **Depleted**



Axial slice through the FHR core

Approach

Calculations performed with the SCALE code system and ENDF/B-VII.1 or ENDF/B-VIII.1 data, plus BNL-LANL-ORNL (BLO) uncertainty data

1. Spent and equilibrium core fuel inventory:

- TRITON reactor physics sequence
 - KENO-VI Monte Carlo neutron transport
 - ORIGEN depletion and decay solver

2. Uncertainty and sensitivity analysis:

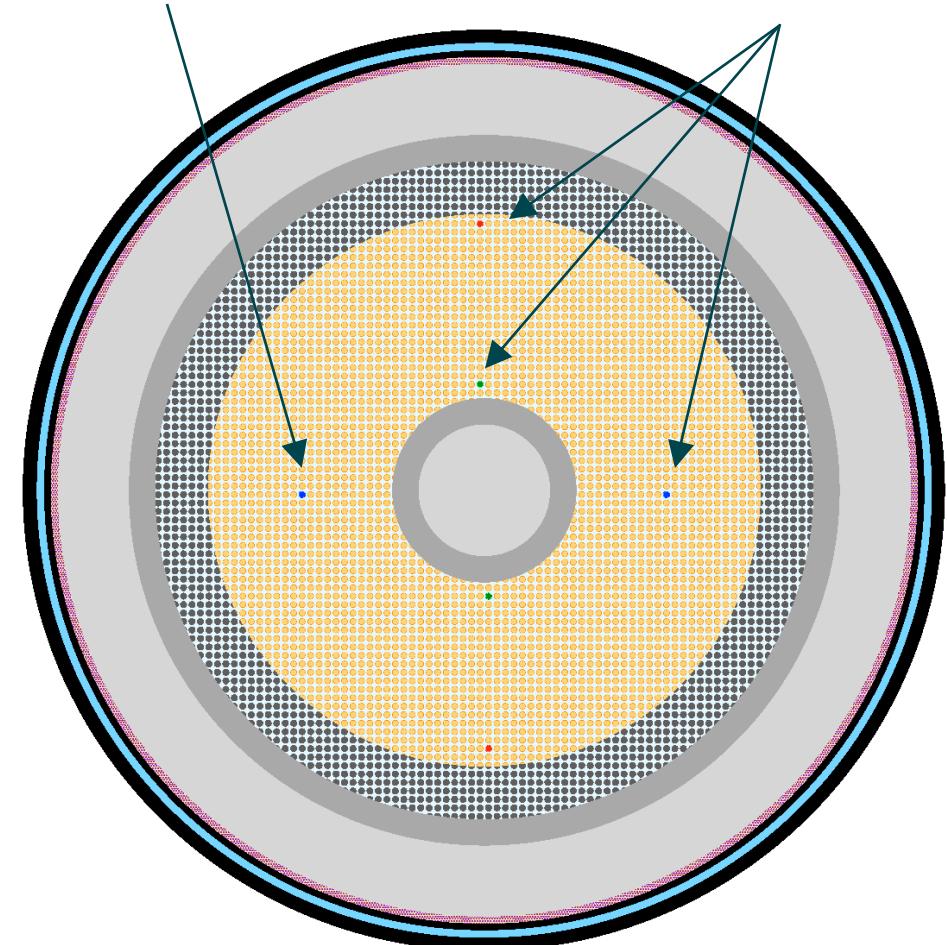
- Sampler uncertainty and sensitivity analysis sequence
- Importance ranking of cross sections in terms of sensitivity index $R^2 \in [0,1]$

Core-average fuel pebbles:

- Core-average fuel composition
- Not depleted

Fresh fuel pebbles:

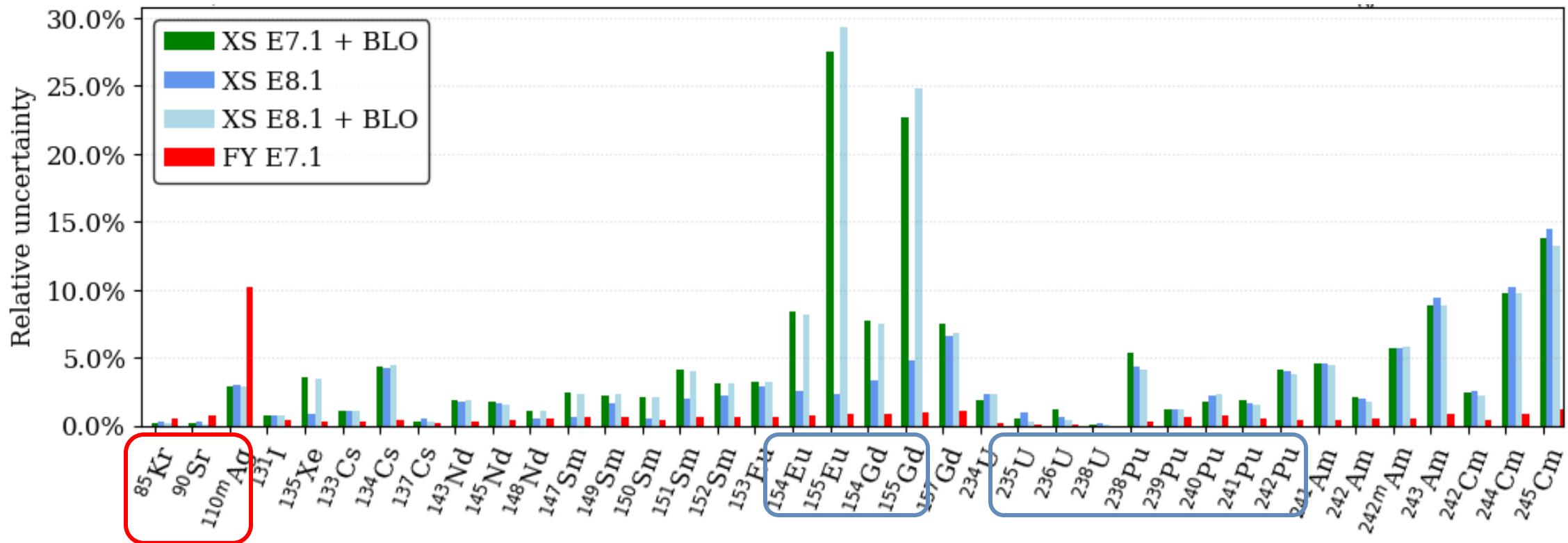
- Fresh fuel composition
- Depleted



Axial slice through the FHR core

Relative uncertainty of spent FHR fuel inventory at discharge

Burnup: 180 GWd/tU



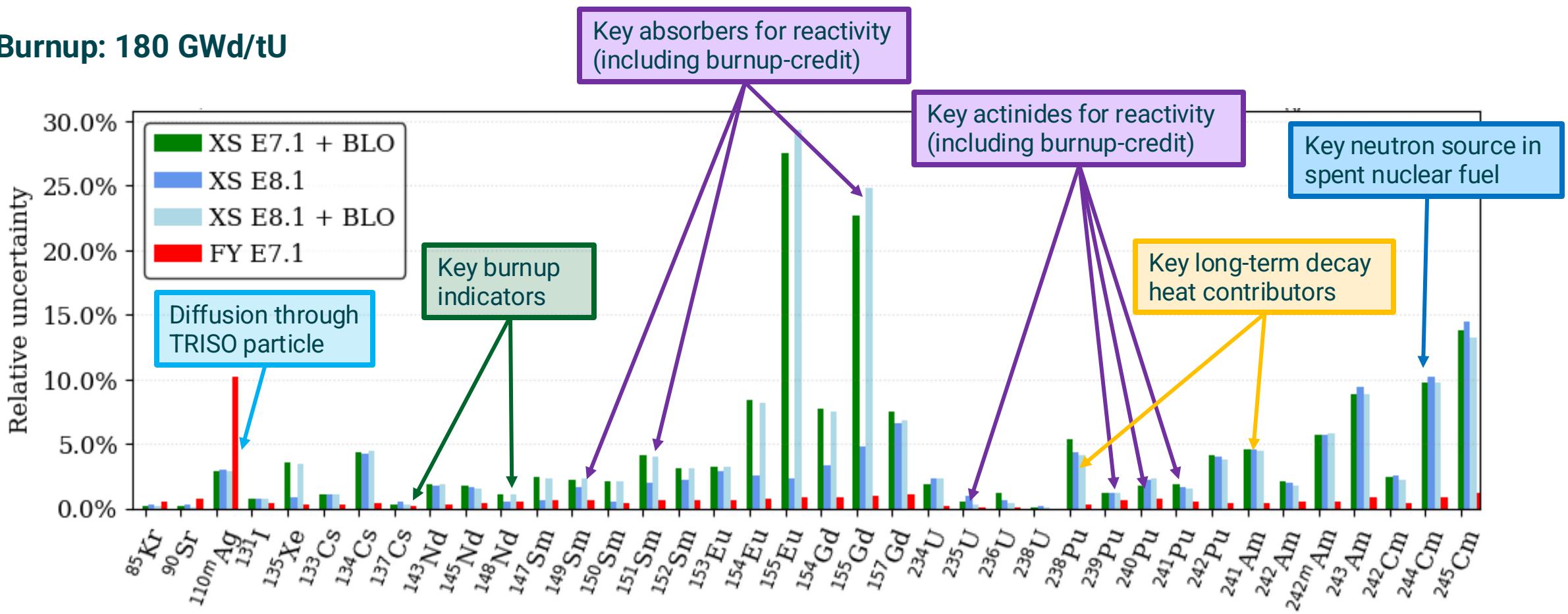
Cross section uncertainties are major drivers for uncertainties, with few exceptions for $\text{Ag}^{110\text{m}}$, Kr^{85} , Sr^{90}

Differences between E8.1+BLO and E7.1+BLO are small, mostly below <1% relative difference

Impact of BLO uncertainties for key fission products Sm, Eu, Gd point to important missing covariance data

FHR fuel inventory uncertainties are relevant for different applications

Burnup: 180 GWd/tU



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Differences between E8.1+BLO and E7.1+BLO are small, mostly below <1% relative difference

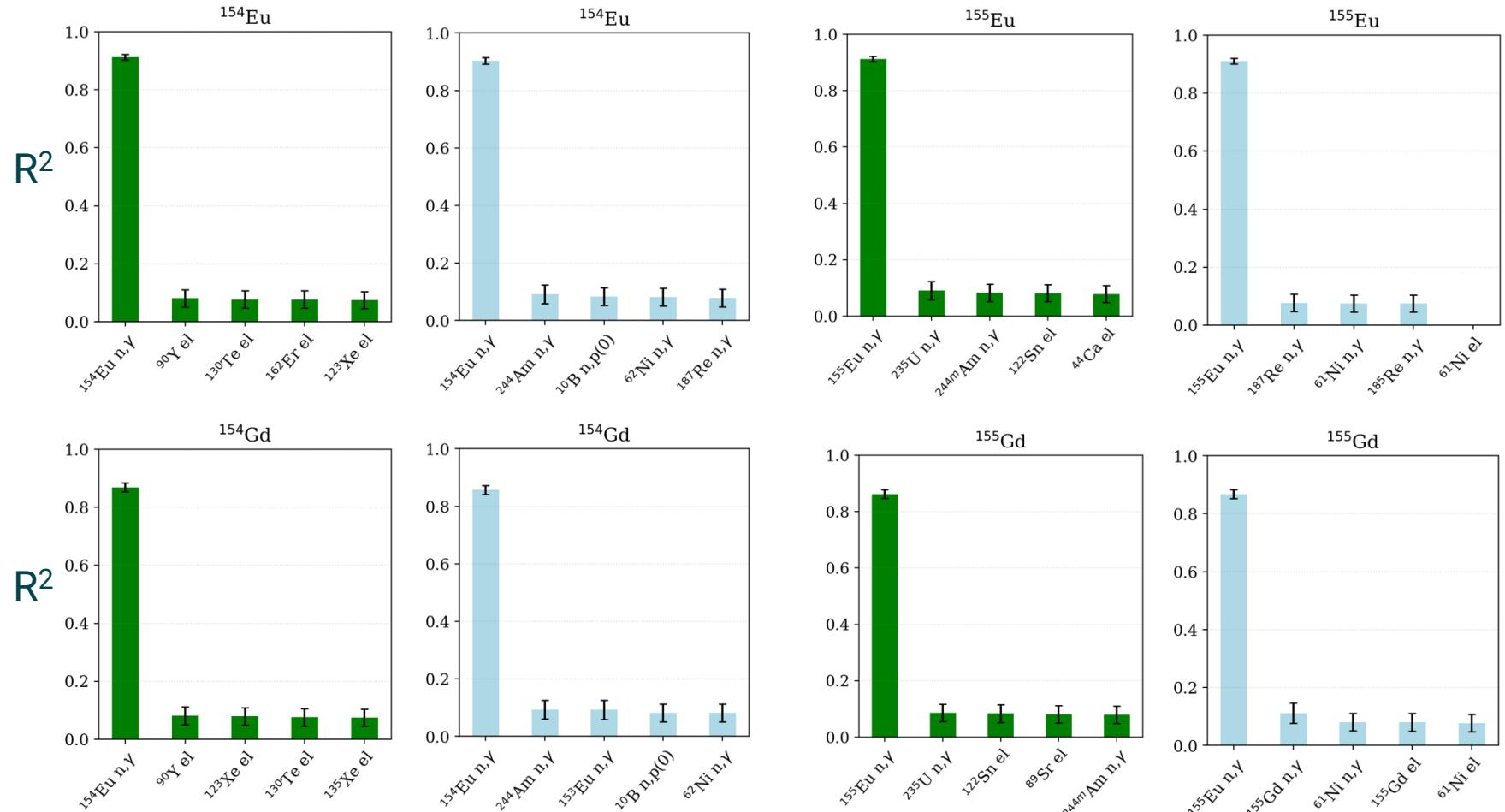
Impact of BLO uncertainties for key fission products Sm, Eu, Gd point to important missing covariance data

Sensitivity analysis reveals missing covariance data influencing important neutron absorber

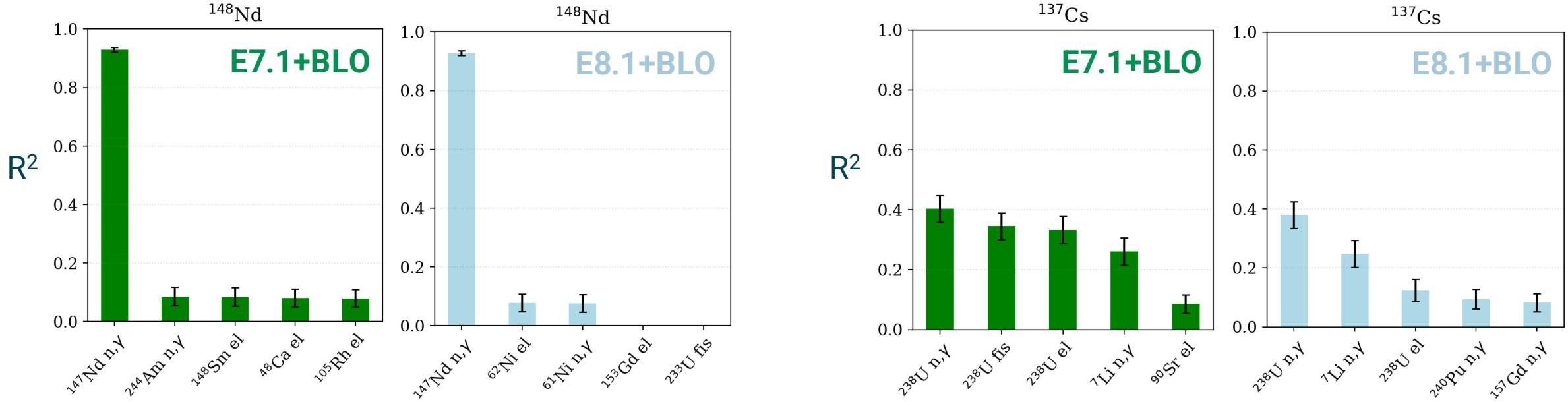
E7.1+BLO

E8.1+BLO

- Uncertainty in Eu-154 (n,γ) is the main contributor to Eu-154 and Gd-154 concentration uncertainty
- Uncertainty in Eu-155 (n,γ) is the main contributor to Eu-155 and Gd-155 concentration uncertainty
- **BUT: Eu-154 (n,γ) and Eu-155 (n,γ) covariance data is not included in E8.1, only in BLO**

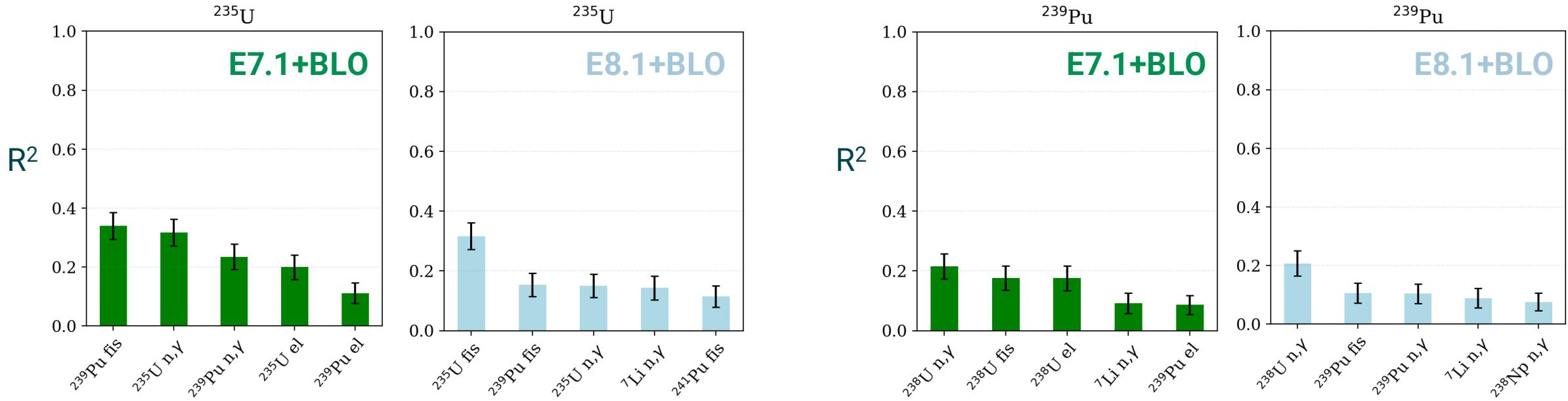


Sensitivity analysis reveals missing and updated covariance data impacting burnup indicators



- Uncertainty in Nd-148 concentration is mainly impacted by Nd-147 (n, γ)
BUT: Nd-147 (n, γ) covariance data is not included in E8.1, only in BLO
- Uncertainty in Cs-137 concentration (0.3%-0.5%) showed small changes between libraries, mainly due to updates in U-238 (n, γ) and fission

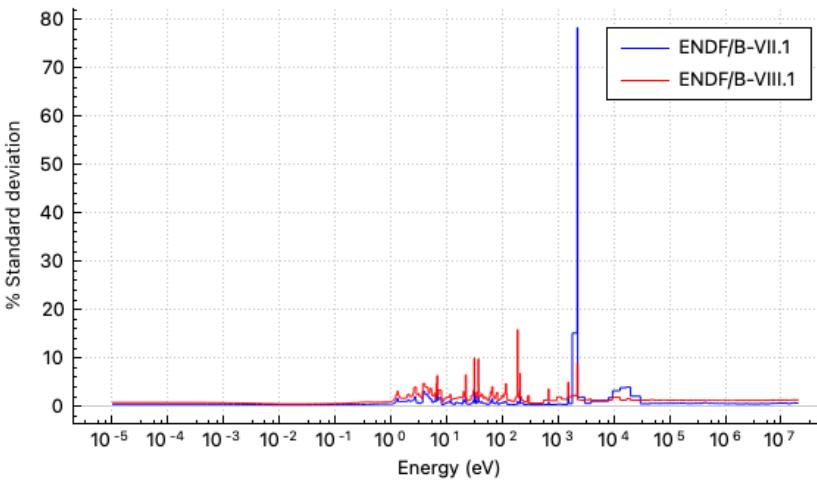
Sensitivity analysis reveals covariance data updates impacting major actinides U-235 and Pu-239



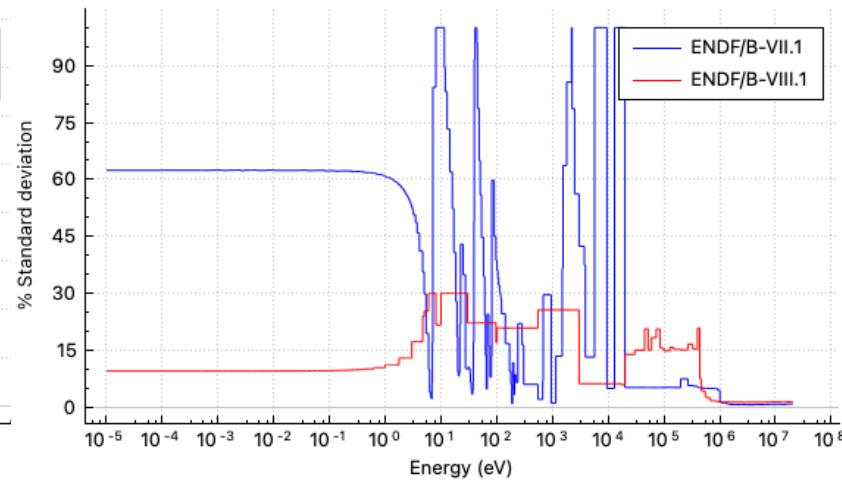
- Uncertainty in U-235 concentration (0.4%-0.5%) shows only small change between libraries, likely due to compensating effects from updates of (n,γ) and fission in U-235 and Pu-239
- Uncertainty in Pu-239 concentration (1.2%) did not change between libraries; but individual uncertainty contributions changed, mainly due to updates in U-238 (n,γ) and fission

Relevant cross section uncertainty updates between E7.1 and E8.1

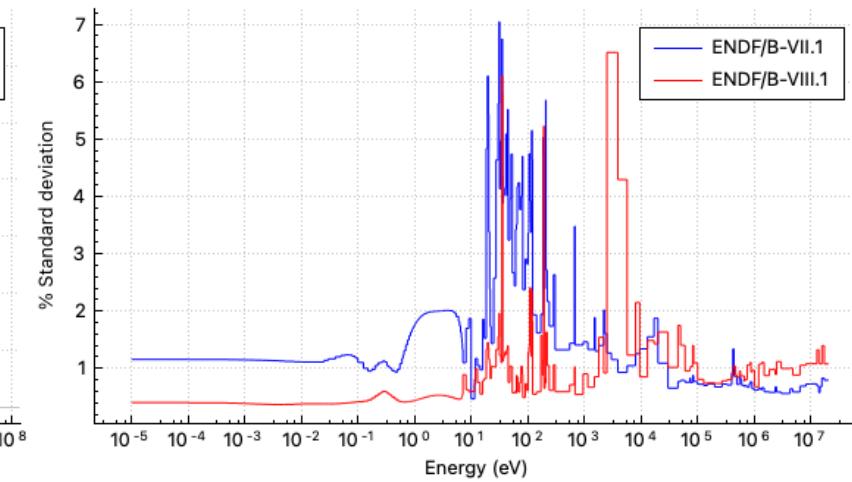
U-235 mt=18 fission



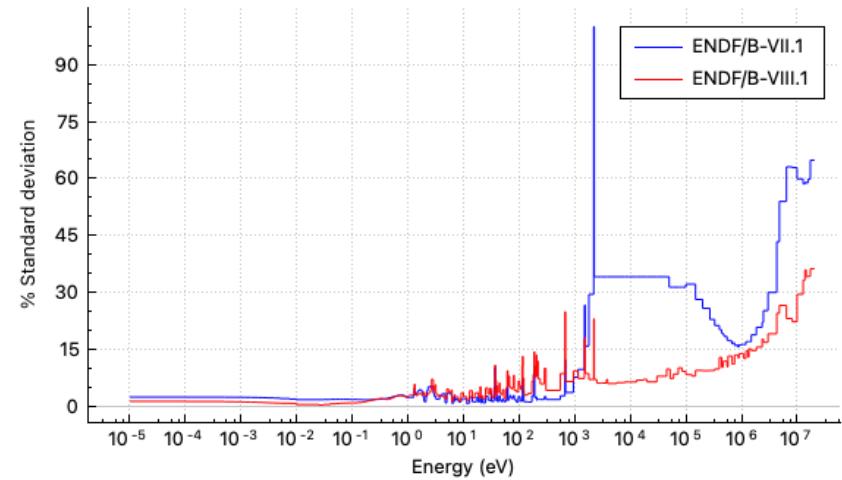
U-238 mt=18 fission



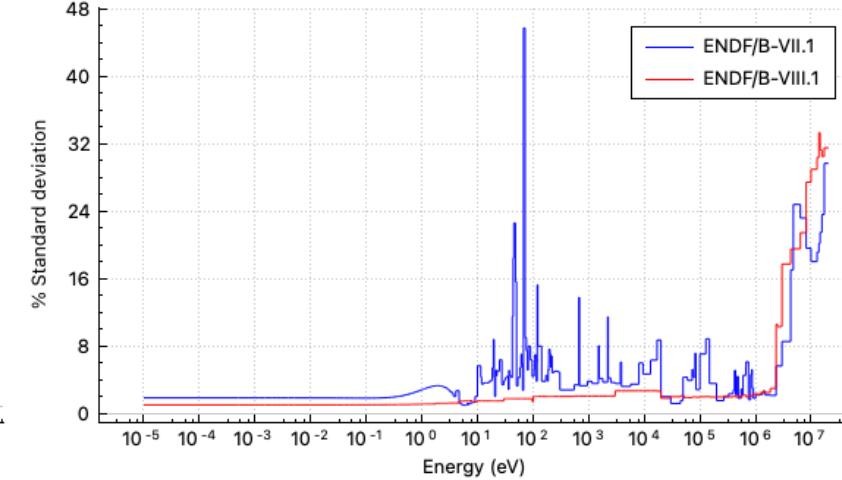
Pu-239 mt=18 fission



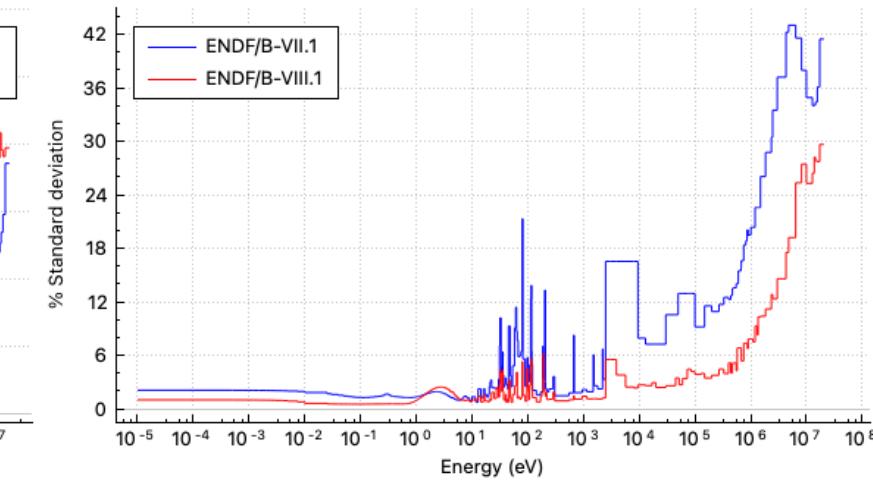
U-235 mt=102 n, gamma



U-238 mt=102 n, gamma



Pu-239 mt=102 n, gamma



Summary and Conclusions

Random sampling approach was applied to investigate the impact of uncertainties in cross sections and fission yields on uncertainties in FHR spent fuel pebble inventory

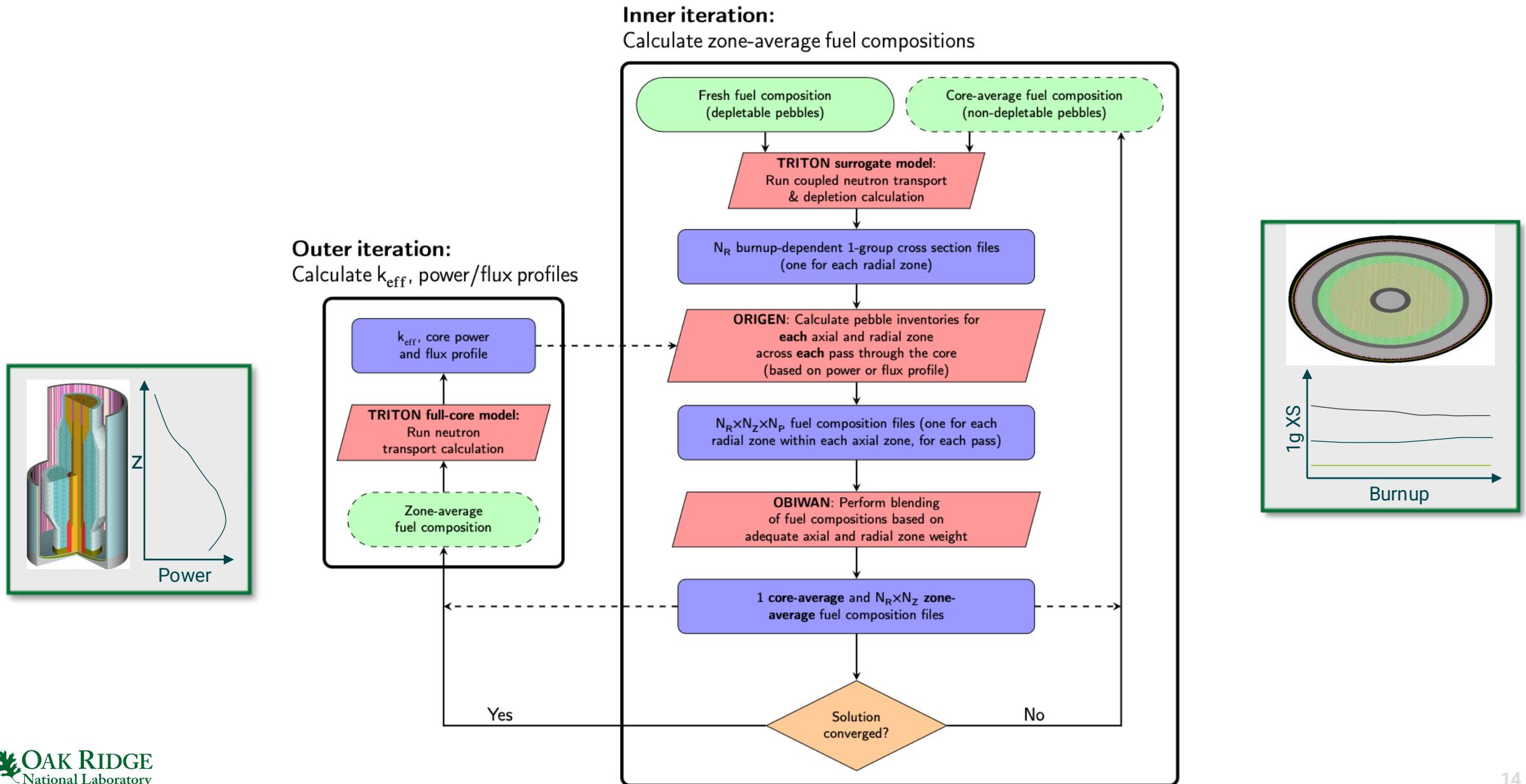
- Fission product yield uncertainties have minor impact, with few exceptions
- Cross section uncertainties cause inventory uncertainties of up to 25% (Sm, Eu, Gd)
- Comparison of cross section uncertainties between libraries highlight:
 - Impact of covariance data update in ENDF/B-VIII.1: U-235, U-238, and Pu-239 (n,γ) and fission
 - Missing important covariance data ENDF/B-VIII.1, as identified through use of BLO data: Nd-148 (n,γ) and Eu-155 (n,γ)
- *Preliminary results presented, with the more comprehensive analysis to follow soon*

Sensitivity analysis identified cross sections that are top contributors of uncertainties in fuel inventory predictions:

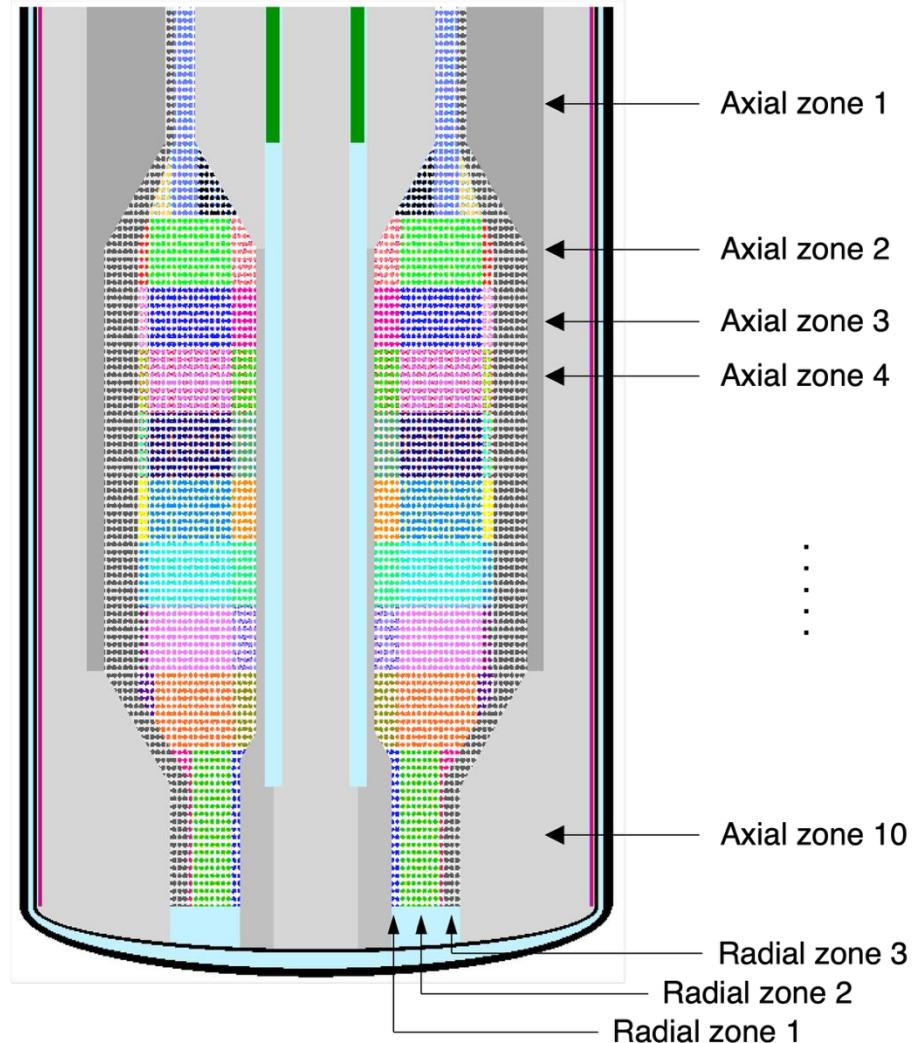
- Change in cross section importance points to relevant cross section uncertainty updates between ENDF/B libraries
- Use of BLO-data reveals impact of cross section covariance data missing in ENDF/B-VIII.1
- Identified top contributing cross sections can inform recommendations for new measurements and evaluations

Backup

SLICE method for PBR equilibrium core generation



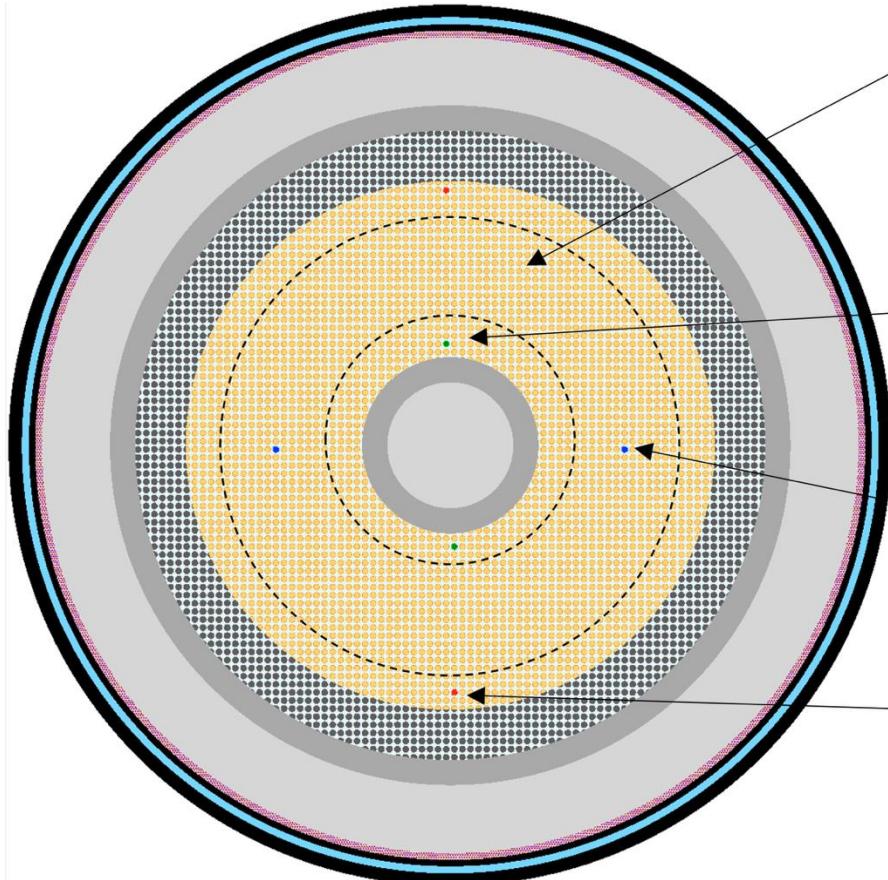
Discretized full core model



Example FHR full core with
discretization of fuel region

- Model discretization based on radial and axial flux profile
- Each region (i.e. specific radial zone within specific radial zone) has unique region-average fuel inventory
- Region-average fuel inventory represents average of fuel pebbles at different passes, at different burnups in this region
- TRITON calculation of this model results in axial/radial flux/power profiles

Axial slice surrogate model



Example FHR axial slice model

Core-average fuel pebbles:

- Core-average fuel composition
- Fuel mixture id 100
- Not to be depleted
- Compositions updated after each inner iteration

Fresh fuel pebble:

- Fresh fuel composition
- Fuel mixture id 150
- Located in innermost radial zone
- To be depleted

Fresh fuel pebble:

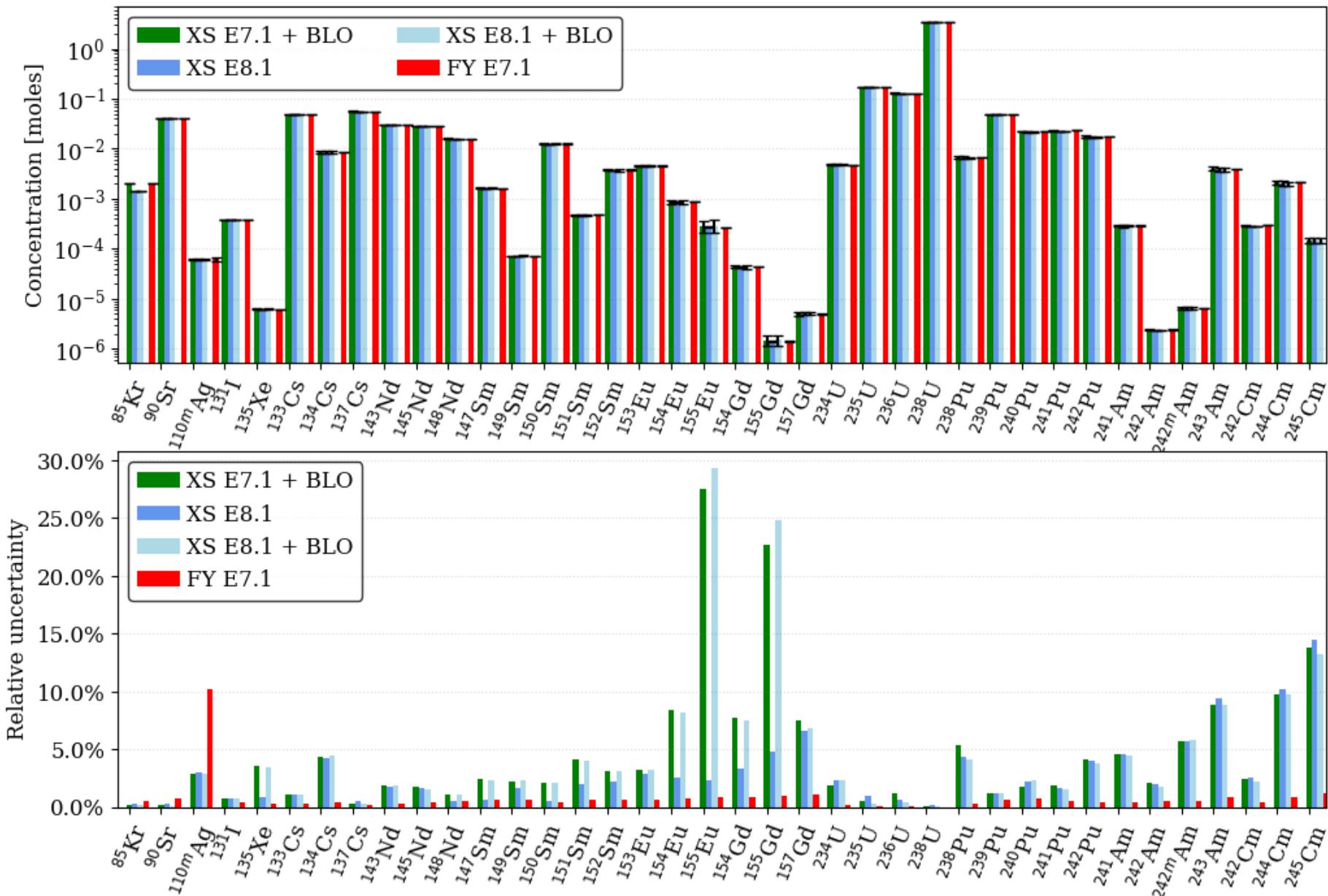
- Fresh fuel composition
- Fuel mixture id 160
- Located in middle radial zone
- To be depleted

Fresh fuel pebble:

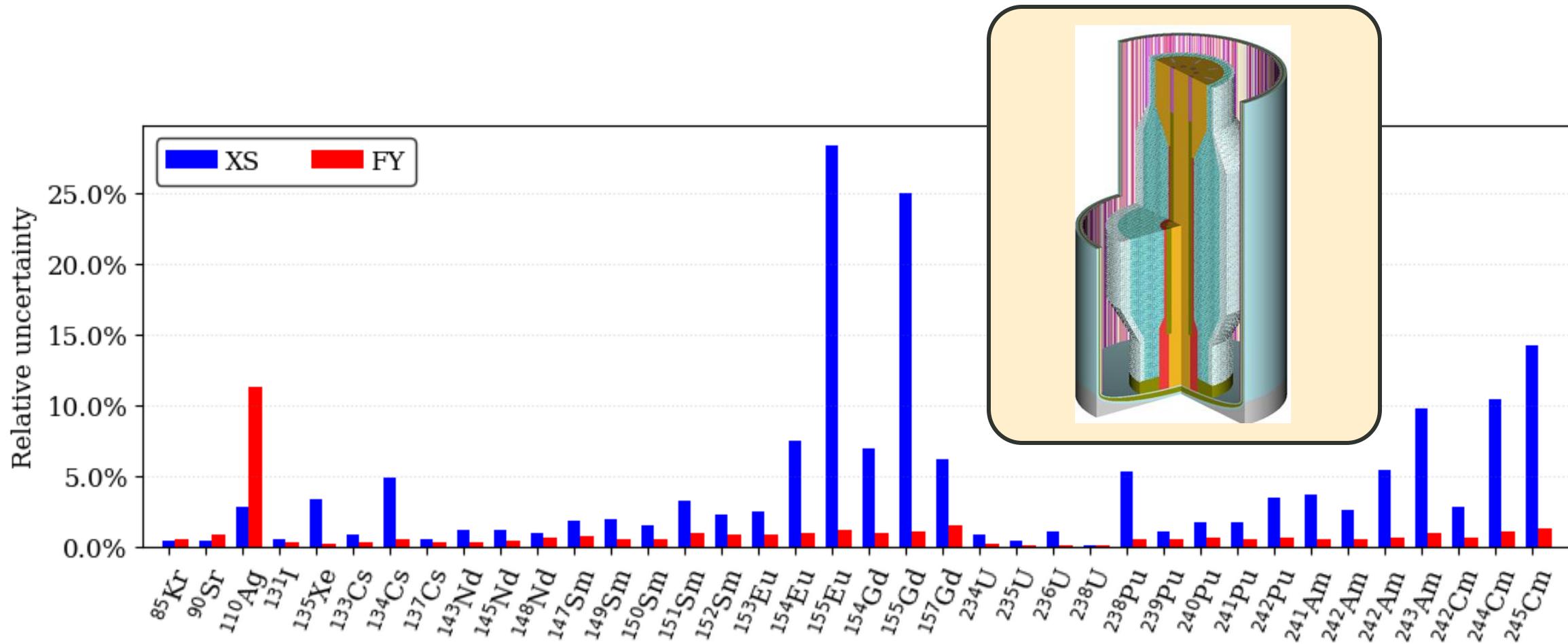
- Fresh fuel composition
- Fuel mixture id 170
- Located in outermost radial zone
- To be depleted

- TRITON model of axial slice through center of the core
- Depletion of only “depletable” fuel pebbles to target discharge burnup
- “Core-average” fuel pebbles provide representative spectral conditions
- TRITON depletion calculation of this model results in ORIGEN cross section libraries (1-group cross sections)

Inventory of a discharged FHR fuel pebble



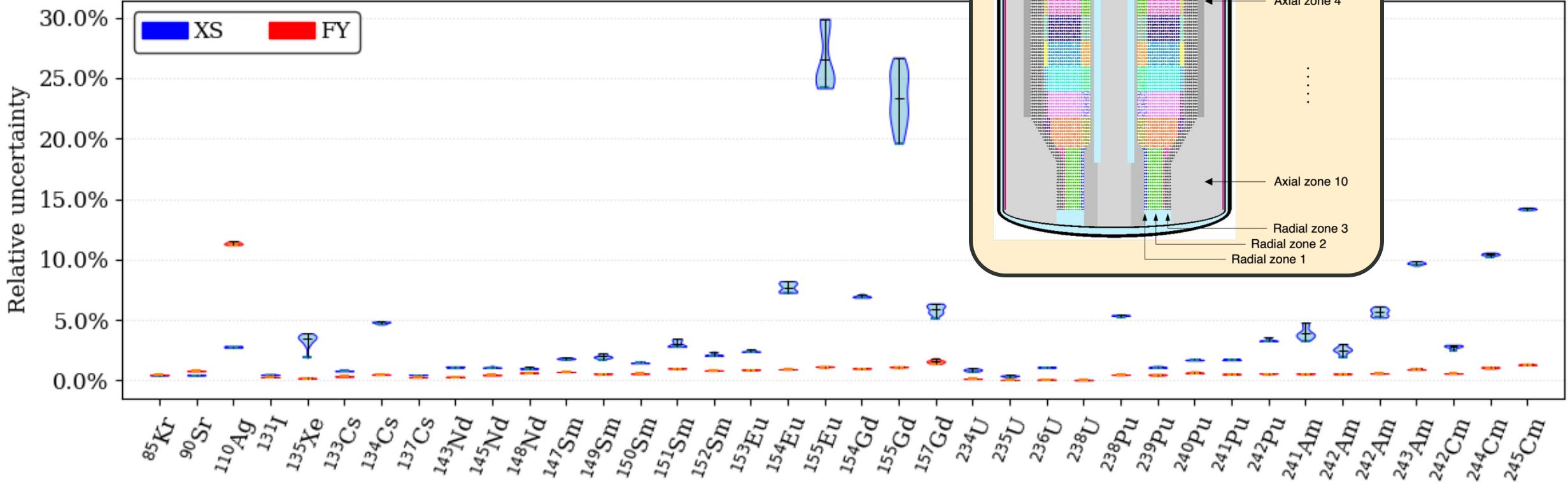
Core-average FHR inventory uncertainty at equilibrium operation using ENDF/B-VII.1



Uncertainties are approximately the same as for the spent fuel inventory

Zone inventory uncertainty at equilibrium operation using ENDF/B-VII.1

Spread of uncertainty in the 30 spatial zones of the core at equilibrium operation



Significant uncertainty spread for only few nuclides: Eu, Gd, Cm isotopes