

# Recent Testing of ENDF/B-VIII.0 at ORNL

W.J. Marshall, T.M. Greene, and  
F. Bostelmann

CSEWG Validation Committee  
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# Outline

1. Overview of comparison of ENDF/B-VII.1 and ENDF/B-VIII.0
2. Summary of results from VALID
3. Detailed comparisons
  - LCT recap
  - MCT/MST/PST individually and combined
4. A few words about graphite
5. Summary

# Overview of comparisons of ENDF/B-VII.1 and ENDF/B-VIII.0

- Use VALID suite to test >600 cases with ENDF/B-VII.1 and ENDF/B-VIII.0 in KENO V.a/KENO-VI
  - SCALE 6.3 beta releases have both libraries
  - Compare  $k_{\text{eff}}$  C/E values
- Limited number of cases tested with TSUNAMI
  - Compare sensitivities primarily through  $c_k$  and  $E$  calculations
  - No further details in this presentation
    - All  $c_k$  values 0.98 or higher
    - All  $E$  values 0.99 or higher

# KENO V.a results summary

Category	ENDF/B-VIII.0		ENDF/B-VII.1		Difference		
	Avg. C/E	Avg. C/E unc.	Avg. C/E	Avg. C/E unc.	(E8.0 – E7.1)	Unc.	$\Delta$ /Unc.
HMF	1.00179	0.00039	1.00198	0.00039	-0.00019	0.00055	0.3
HST	0.99824	0.00074	0.99774	0.00072	0.00050	0.00103	0.5
IMF	1.00061	0.00082	1.00289	0.00083	-0.00228	0.00117	2.0
LCT	0.99921	0.00018	0.99960	0.00018	-0.00039	0.00025	1.5
LST	0.99845	0.00083	0.99823	0.00083	0.00022	0.00117	0.2
MCF	0.99797	0.00157	0.99890	0.00157	-0.00093	0.00222	0.4
MCT	0.99811	0.00087	0.99916	0.00087	-0.00105	0.00123	0.9
MST	0.99354	0.00157	0.99839	0.00158	-0.00485	0.00223	2.2
PMF	0.99942	0.00062	0.99952	0.00062	-0.00010	0.00088	0.1
PST	0.99772	0.00055	1.00301	0.00056	-0.00529	0.00078	6.7
UCT	0.99818	0.00140	1.00080	0.00141	-0.00262	0.00199	1.3
UMF	0.99860	0.00051	0.99845	0.00051	0.00015	0.00072	0.2
USI	0.97945	0.00123	0.98275	0.00124	-0.00330	0.00175	1.9
USM	0.97546	0.00214	0.97901	0.00215	-0.00355	0.00303	1.2
UST	0.99750	0.00052	1.00016	0.00052	-0.00266	0.00074	3.6

# KENO-VI results summary

Category	ENDF/B-VIII.0		ENDF/B-VII.1		Difference		
	Avg. C/E	Avg. C/E unc.	Avg. C/E	Avg. C/E unc.	(E8.0 – E7.1)	Unc.	$\Delta$ /Unc.
HMF	0.99814	0.00044	0.99872	0.00044	-0.00058	0.00062	0.9
IMF	1.00407	0.00274	1.00589	0.00275	-0.00183	0.00388	0.5
MCT	0.99362	0.00078	0.99417	0.00078	-0.00055	0.00110	0.5

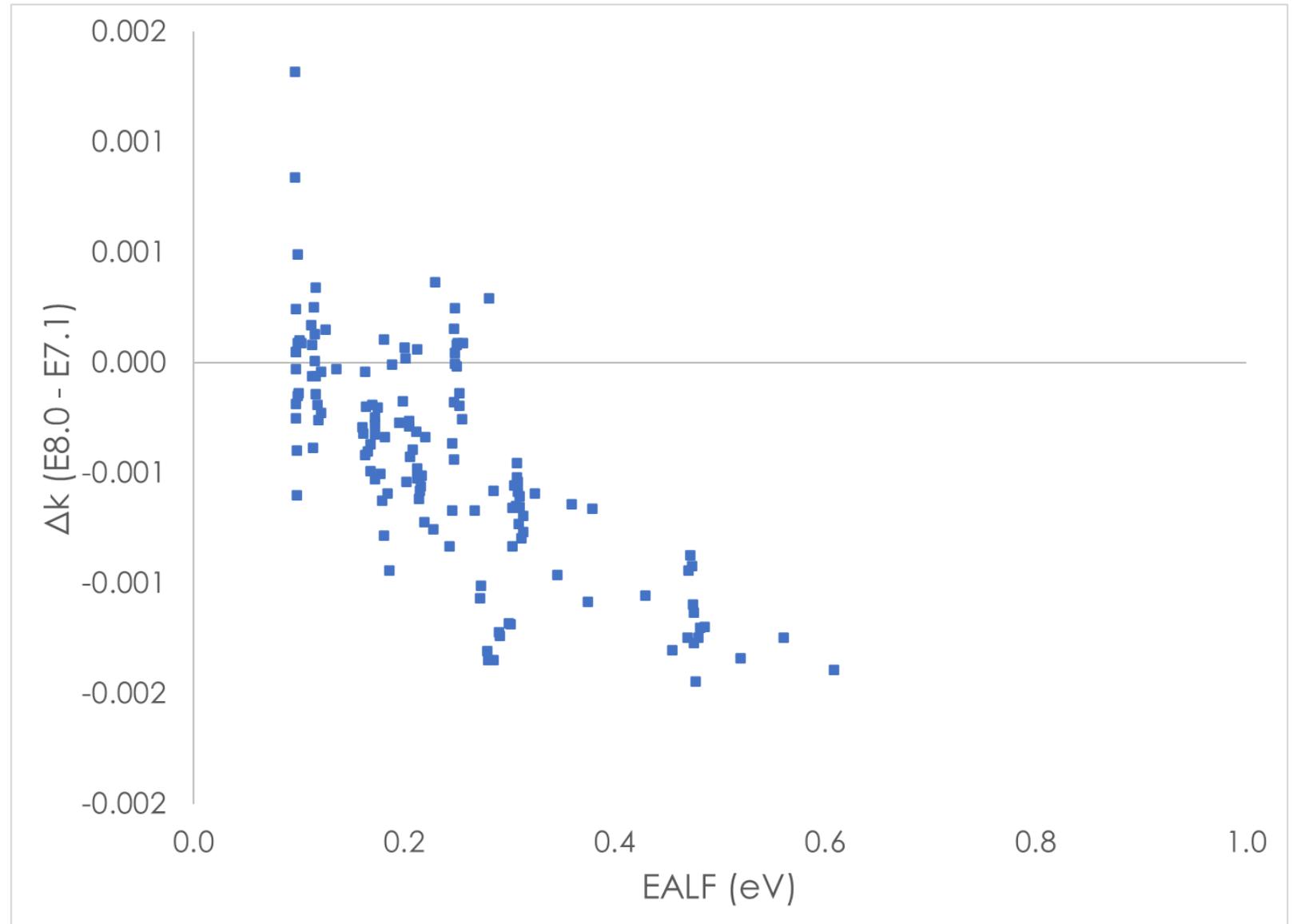
# Pooled results summary

Category	ENDF/B-VIII.0		ENDF/B-VII.1		Difference		
	Avg. C/E	Avg. C/E unc.	Avg. C/E	Avg. C/E unc.	(E8.0 – E7.1)	Unc.	$\Delta$ /Unc.
HMF	0.99982	0.00030	1.00022	0.00030	-0.00040	0.00042	1.0
IMF	1.00114	0.00081	1.00335	0.00082	-0.00222	0.00115	1.9
MCT	0.99555	0.00058	0.99631	0.00058	-0.00076	0.00082	0.9

- HMF: 23 KENO V.a and 27 KENO-VI (50 total cases)
- IMF: 11 KENO V.a and 2 KENO-VI (13 total cases)
- MCT: 21 KENO V.a and 28 KENO-VI (49 total cases)
- Such pooling not recommended for code validation

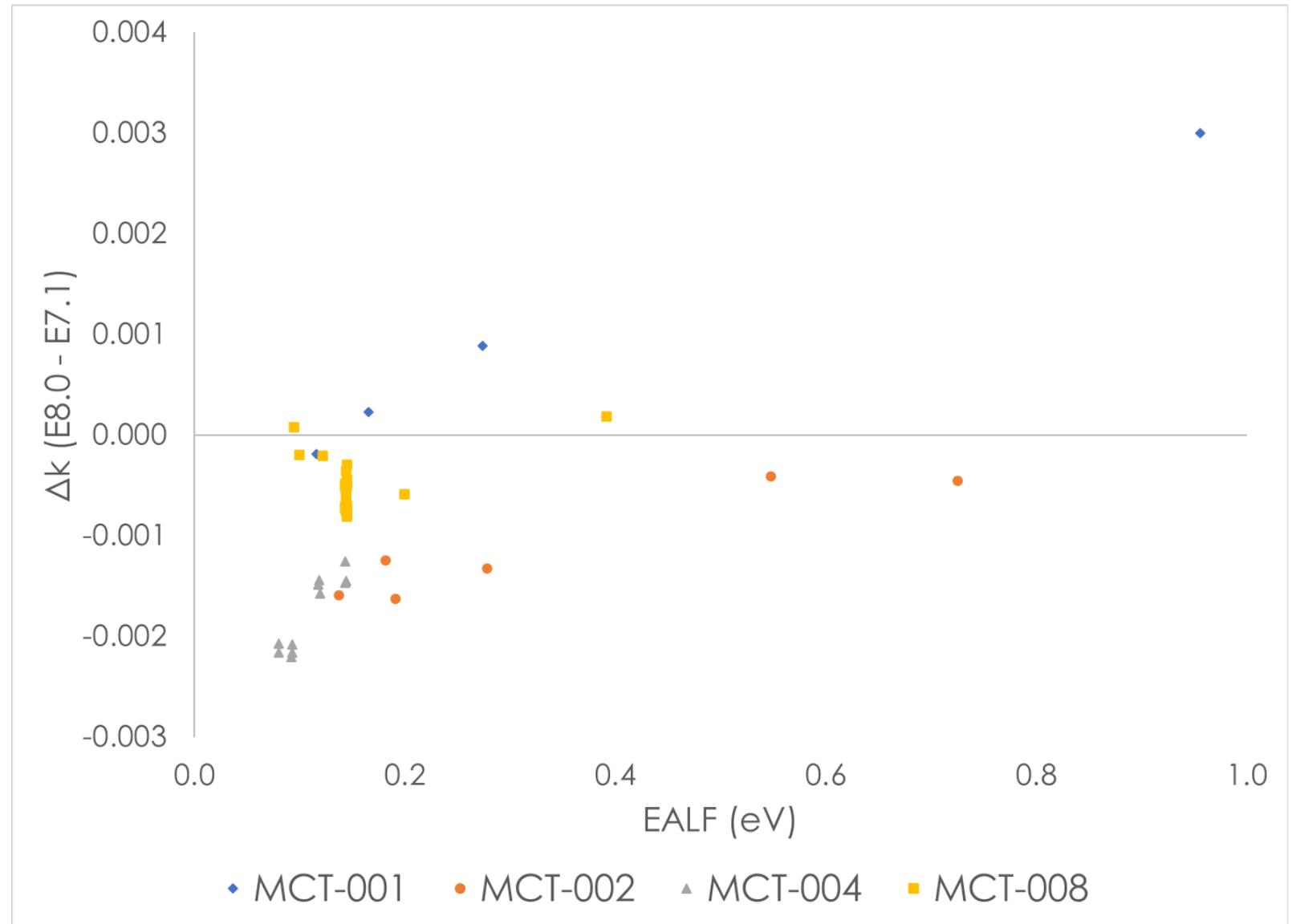
# Changes for LCT experiments – recap from last year

- Bias attributed to  $^{16}\text{O}$
- Lower calculated  $k_{\text{eff}}$  values represent increased magnitude bias for ENDF/B-VIII.0



# Changes for MCT experiments

- Slightly positive trend
  - Opposite of LCTs
- Lower calculated  $k_{\text{eff}}$  values represent increased magnitude bias for ENDF/B-VIII.0



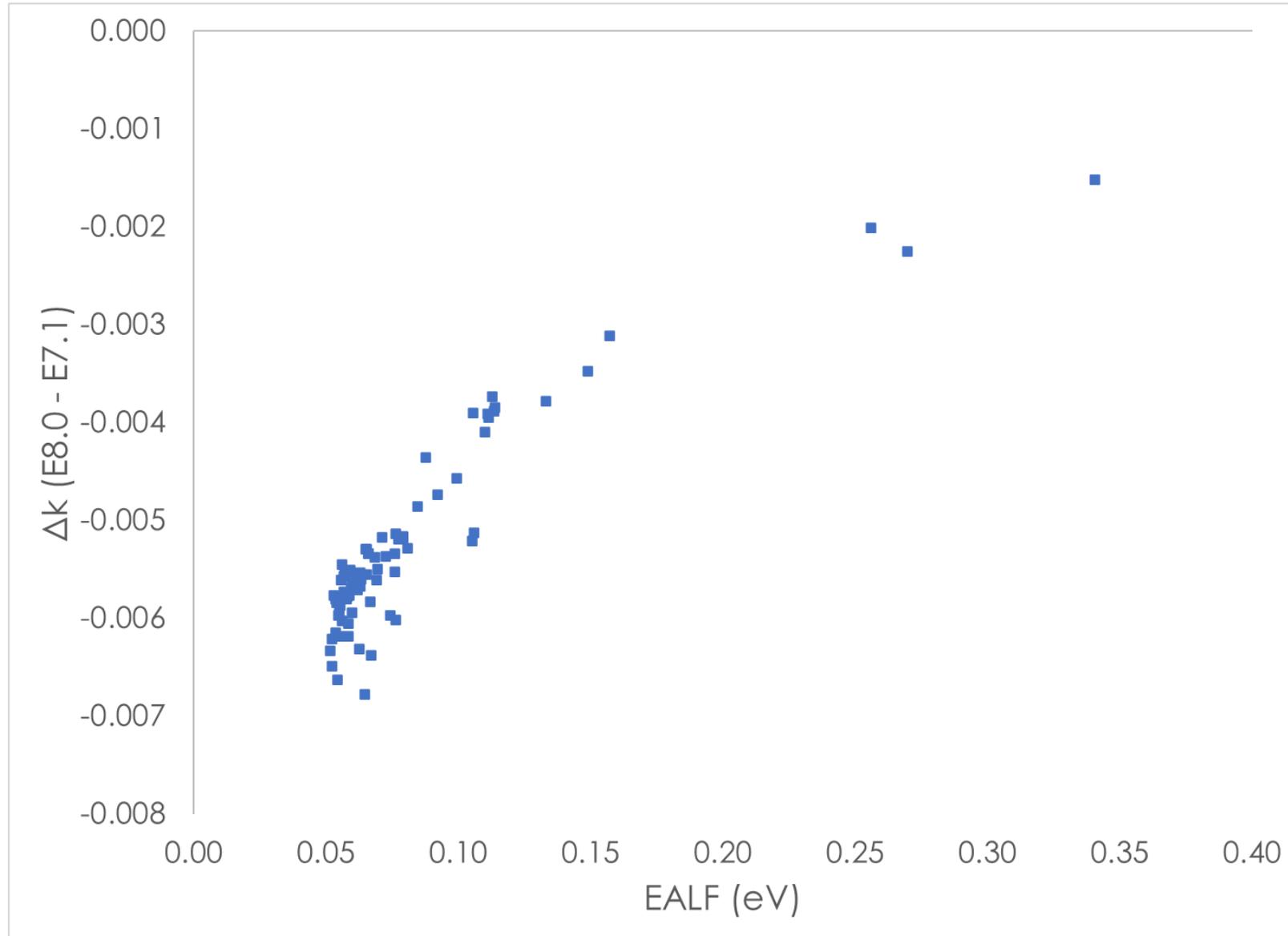
# Changes for MST experiments

- MCT delta was unexpected
- What trend do MST experiments show?
- Lower calculated  $k_{\text{eff}}$  values represent increased magnitude bias for ENDF/B-VIII.0
- Only 10 cases, but significantly lower average C/E:
  - ENDF/B-VII.1:  $0.99839 \pm 0.00158$
  - ENDF/B-VIII.0:  $0.99354 \pm 0.00157$
- Consistent with PST results
  - Natural or depleted uranium in all cases

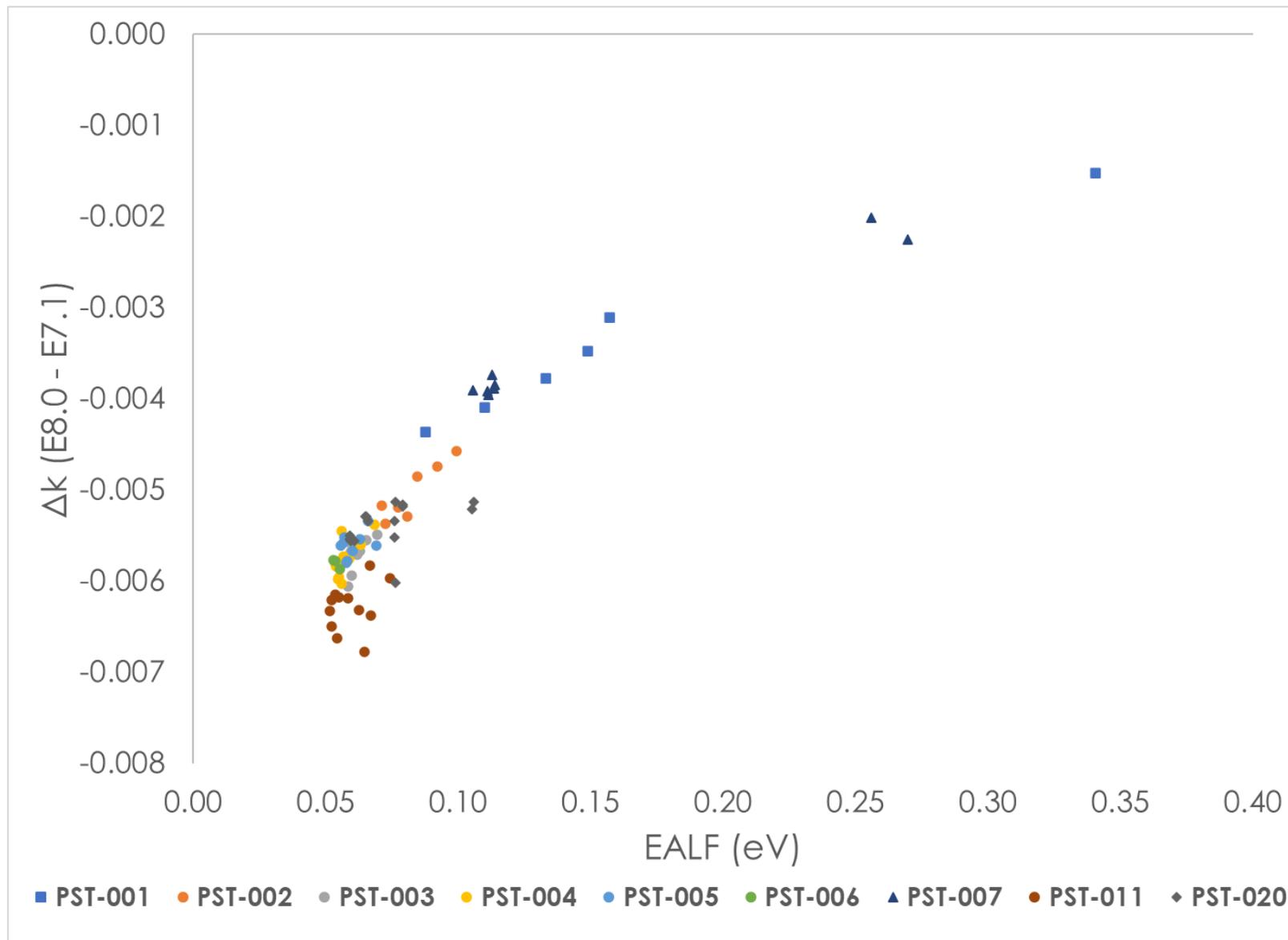
# Changes for PST experiments

- Decreasing overprediction for PST experiments is one of the top-line features of ENDF/B-VIII.0
- Average C/E dropped from  $1.00301 \pm 0.00056$  to  $0.99772 \pm 0.00055$
- Change not uniform – as shown on next slide
- Generally consistent trend across multiple evaluations

# Changes for PST experiments (continued)

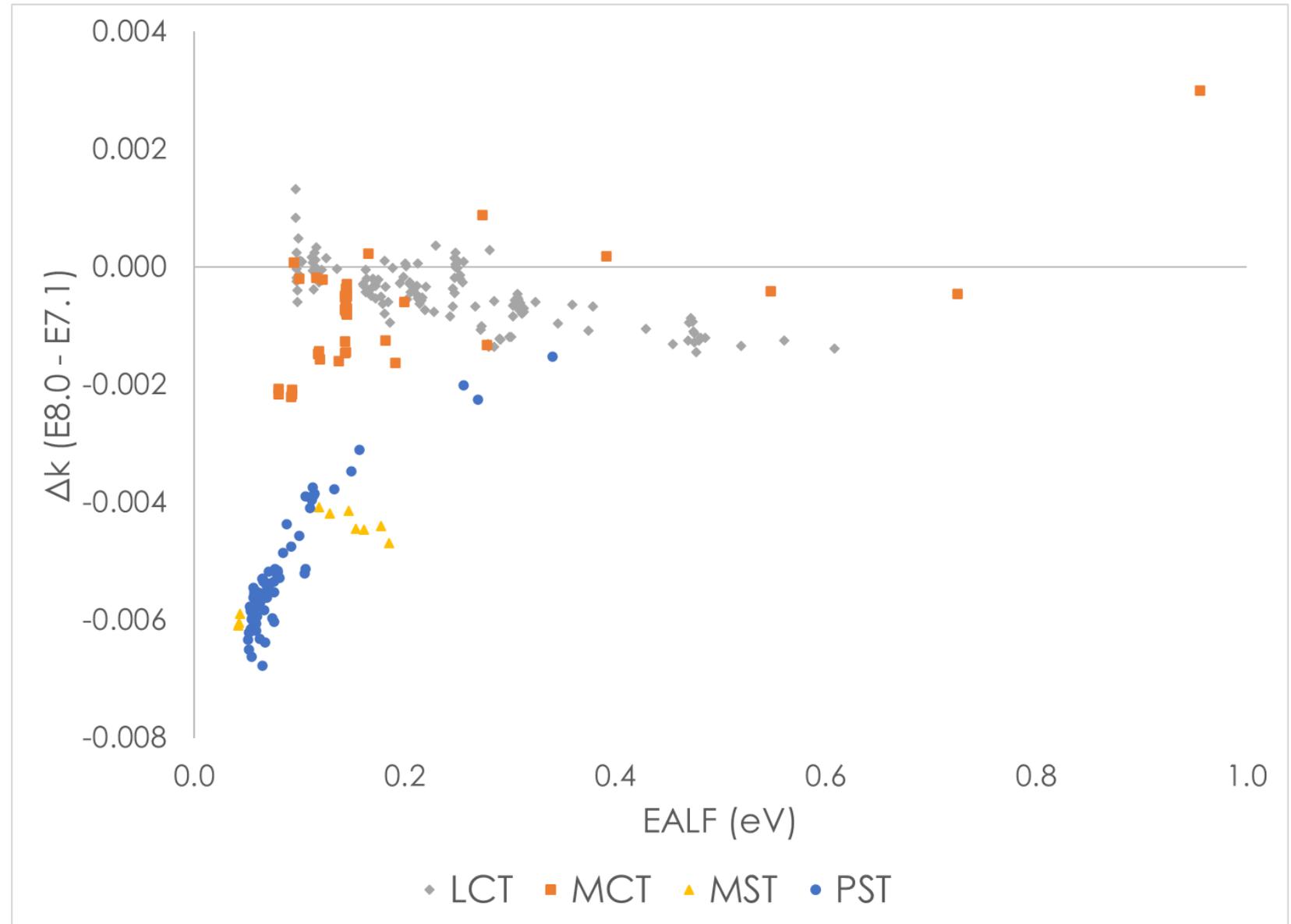


# Changes for PST experiments (continued)



# Crossing the streams...

- MCT results driven by Pu
- Predictions of oxide-fueled array experiments closer to measurements than solutions for highly thermalized systems



# A few words about graphite

- Graphite performance investigated across ENDF/B-VII.0, ENDF/B-VII.1, and ENDF/B-VIII.0 for MSR and HTGR models
  - Results published previously at ANS Winter Meeting 2019 and *Annals of Nuclear Energy* June 2020
  - Models at room temperature
    - Elevated temperature HTTR models may be examined in the future
- Brief summary of the results presented here to make sure the Validation Committee is aware of the work

# Eigenvalue comparison – graphite moderated systems

	$\Delta\rho$ (pcm) ENDF/B-VII.0 vs. ENDF/B-VII.1	$\Delta\rho$ (pcm) ENDF/B-VIII.0 vs. ENDF/B-VII.1
Graphite moderated MSR cell – fresh	259(10)	-68(22)
Graphite moderated MSR cell – depleted	483(22)	-93(24)
HTR (Full core model)	913(16)	-312(19)
HTR-10 (Full core model)	1090(14)	-311(13)

Carbon capture update,  
e.g. at 0.0253 eV:

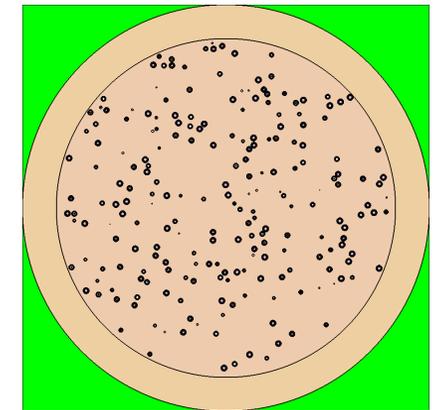
- ENDF/B-VII.0: 3.368 mb
- ENDF/B-VII.1: 3.861 mb

Differences  
caused by?

# HTR-10 pebble model: ENDF/B-VII.1 vs. ENDF/B-VIII.0

Replace individual nuclides in ENDF/B-VII.1 calculation by ENDF/B-VIII.0 data:

<b>Basis: ENDF 7.1</b>	<b><math>\Delta\rho</math> to all ENDF 7.1 (pcm)</b>
But: graphite from ENDF 8.0	<b>-3(14)</b>
But: $^{235}\text{U}$ from ENDF 8.0	<b>-251(13)</b>
But: $^{238}\text{U}$ from ENDF 8.0	<b>85(13)</b>
All ENDF 8.0	<b>-157(13)</b>



**HTR-10 fuel pebble**

- Differences between ENDF/B-VII.0 and VII.1: carbon capture
- Differences between ENDF/B-VII.1 and VIII.0:  $^{235}\text{U}$  and  $^{238}\text{U}$

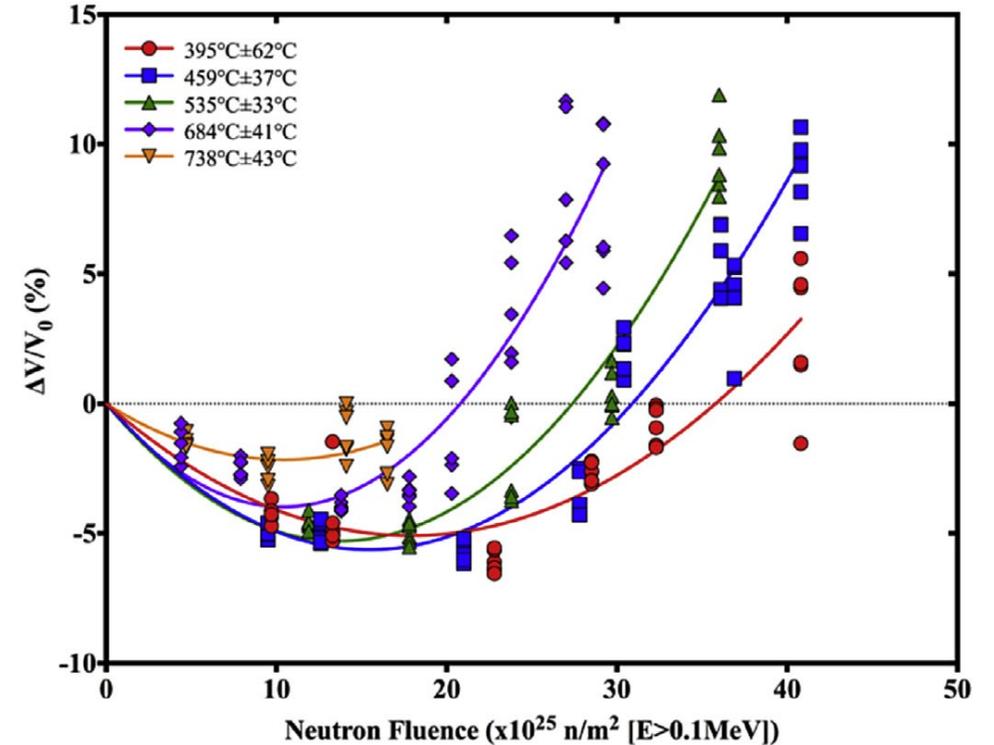
# ENDF/B-VIII.0: graphite with different porosities

## ENDF/B-VIII.0 provides different graphite data:

1. perfect crystal
2. 10% porosity
3. 30% porosity

### HTR-10: ENDF/B-VIII.0 data

Applied graphite data	$\Delta\rho$ (pcm)
All graphite	(ref)
10% porosity in reflector (but not in pebbles)	<b>1(49)</b>
30% porosity in reflector (but not in pebbles)	<b>-165(55)</b>
All 10% porosity	<b>342(28)</b>
All 30% porosity	<b>665(26)</b>



**Specimen volume change versus neutron fluence for specimens irradiated at different temperatures (courtesy of A. A. Campbell)**

Do we always know our porosity? In each area?

Anne A. Campbell, et al., "Property Changes of G347A Graphite Due to Neutron Irradiation," *Carbon* 109, pp. 860–873, 2016.

# Summary

- ORNL has recently completed a comparison of ENDF/B-VIII.0 and ENDF/B-VII.1 using the VALID library
  - ORNL/TM report in review, to be published soon
- Work is also being performed for advanced reactor applications
- Other efforts on-going
  - Covariance data evaluations discussed in covariance committee
  - Expanding VALID for greater applicability to advanced reactors
    - Intermediate enrichment
    - Graphite and lead
  - Also trying to investigate more intermediate spectrum cases

# Questions?

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